

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Dresden Nuclear Power Station	DOCKET NUMBER (2) 0 5 0 0 0 0 2 3 7	PAGE (3) 1 OF 0 2
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TITLE (4)  
Failure of Primary Containment Type "B" and "C" Leak Testing

EVENT DATE (6)			LER NUMBER (8)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (9)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		
									N/A		
1	2	1	1	8	4	8	4		N/A		
				0	2				DOCKET NUMBER(S) 0 5 0 0 0 0		
									DOCKET NUMBER(S) 0 5 0 0 0 0		

OPERATING MODE (9)  N

POWER LEVEL (10) 0 0 0

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 50. (Check one or more of the following) (11)

20.402(b)	20.408(e)	90.73(a)(2)(iv)	73.71(b)
20.408(a)(1)(i)	90.36(a)(1)	90.73(a)(2)(v)	73.71(e)
20.408(a)(1)(ii)	90.36(a)(2)	90.73(a)(2)(vi)	OTHER (Specify in Abstract below and in Text, NRC Form 365A)
20.408(a)(1)(iii)	90.73(a)(2)(i)	90.73(a)(2)(vii)(A)	
20.408(a)(1)(iv)	90.73(a)(2)(ii)	90.73(a)(2)(vii)(B)	
20.408(a)(1)(v)	90.73(a)(2)(iii)	90.73(a)(2)(ix)	

LICENSEE CONTACT FOR THIS LER (12)

NAME Lawrence Coyle	TELEPHONE NUMBER (483) 8 1 5 9 4 2 - 2 9 2 0
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COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC
X	B	O	V C	6 8 4	Y				

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE)  NO

EXPECTED SUBMISSION DATE (15)

MONTH	DAY	YEAR
0	3	2 9 8 5

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

During the unit refueling outage, while performing DTS 1600-1 (Local Leak Rate Testing of Primary Containment Valves), the torus to condenser drain valve AO 2-1599-61 leaked 121 SCFH. This brings the total as found through leakage for type "B" and "C" testing to 542 SCFH, which exceeds the Technical Specifications limit of 493.116 SCFH. Safety significance was minimal since the other in line isolation valve, AO 2-1599-62, showed no leakage and was capable of isolating the line. Previous occurrence of a failure of type "B" and "C" leak testing was reported by R.O. 84-19 on Docket 50-249.

Cause of the event is unknown at this time. The valve will be repaired and leak rate tested prior to returning it to service. All other primary containment isolation valves which failed the station's procedural limits for leakage which is greater than 31 SCFH, will also be repaired prior to returning them to service; lowering the as left through leakage for type "B" and "C" testing well below the Technical Specifications limit.

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PDR ADDCK 05000237  
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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
		8   4	-   0   2   3	-   0   0	0   2	OF

TEXT (If more space is required, use additional NRC Form 388A's) (17)

During the unit refueling outage, while performing DTS 1600-1 (Locak Leak Rate Testing of Primary Containment Isolation Valves), the torus to condenser drain valve AO 2-1599-61 leaked 121 SCFH. This brings the total as found through leakage for type "B" and "C" testing to 542 SCFH, which exceeds the Technical Specifications limit of .6 La or 493.116 SCFH. Safety significance was minimal since the other in-line isolation valve, AO 2-1599-62, showed zero leakage and was capable of isolating the line. A previous occurrence of a failure of type "B" and "C" leak testing was reported by R.O. 84-19 on Docket 50-249.

Cause of the event is unknown at this time. The valve will be repaired and local leak rate tested prior to returning it to service. All other primary containment isolation valves which failed the station's conservative procedural limits for leakage of 31 SCFH, will also be repaired and local leak rate tested prior to returning them to service; lowering the total as left through leakage for type "B" and "C" testing well below the Technical Specifications limit of 493.116 SCFH.



**Commonwealth Edison**  
Dresden Nuclear Power Station  
R.R. #1  
Morris, Illinois 60450  
Telephone 815/942-2920

January 8, 1985

DJS Ltr #85-26

U.S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, D.C. 20555

Licensee Event Report #84-023-0, Docket #050237 is being submitted as required by Technical Specification 6.6, NUREG 1022 and 10 CFR 50.73 (a)(2)(ii).

W.J. Scott  
Station Superintendent  
Dresden Nuclear Power Station

DJS/kjl

Enclosure

cc: J.G. Keppler, Regional Administrator, Region III  
File/NRC  
File/Numerical

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1/1