CP&L

Carolina Power & Light Company

SERIAL: NLS-85-058

MAR 2 2 1985

Director of Nuclear Reactor Regulation
Attention: Mr. D. B. Vassallo, Chief
Operating Reactors Branch No. 2
Division of Licensing
United States Nuclear Regulatory Commission
Washington, DC 20555

BRUNSWICK STEAM ELECTRIC PLANT, UNIT NOS. 1 AND 2
DOCKET NOS. 50-325 & 50-324/LICENSE NOS. DPR-71 & DPR-62
BWR FEEDWATER AND CRD RETURN LINE NOZZLE CRACKING, NUREG-0619

Dear Mr. Vassallo:

By letter dated October 12, 1984, Carolina Power & Light Company (CP&L) committed to provide our schedule to complete the evaluation of the feedwater controller and the benefits of rerouting the reactor water cleanup (RWCU) system piping by March 1985.

The Company contracted General Electric Company (GE) to provide recommendations concerning instrumentation and procedures necessary to provide sufficient operating data to evaluate the possible need for controller modifications and/or RWCU system piping rerouting. Based on these recommendations, it has been determined that the existing instrumentation at Brunswick does not provide sufficient operating data to make the required evaluation.

GE's recommendations will be used to formulate plant modifications which will include instrumentation installation instructions and procedures for data acquisition and usage factor calculations. In addition, the plant modification package will include Technical Specification and FSAR changes as required.

This project has been scheduled in the Long Range Plan, resulting in tentative installation dates of:

Brunswick-1

Refuel Outage No. 5 (presently scheduled to begin in May 1987)

Brunswick-2

Refuel Outage No. 7 (presently scheduled to begin March 1988)

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The units will have to be operated for at least one cycle after the instrumentation is installed in order to collect sufficient operating data to evaluate the feedwater low flow controller and RWCU reroute.

Also, by letter dated February 24, 1983, Carolina Power & Light Company (CP&L) provided our plans to complete cutting and capping of the Brunswick-1 control rod drive (CRD) return line during the upcoming reload 4 outage scheduled to begin March 31, 1985. This item is scheduled for completion during this outage. However, due to a change in the outage conditions, i.e., the in-vessel work previously planned for this outage has been cancelled, the required final liquid penetrant (LP) examination of the inner blend radius cannot be completed during this outage. The Company will perform this required testing during the reload 5 outage. Justification is as follows:

1. No other in-vessel work is scheduled for reload 4 that would require draining the vessel. Performing the final LP test concurrent with the feedwater nozzle work scheduled during reload 5 is consistent with CP&L's corporate ALARA program.

2. The CRD return line has been isolated since 1979 at which time the nozzle inner blend radius was LP tested and the cracks were ground out. Due to the elimination of flow, no additional significant thermal cycling has occurred since the line was isolated. Since the source of thermal fatigue has been removed and the inner blend radius inspected thereafter, crack initiation is not expected.

The Company believes that this delay in performing the LP testing will not affect the health or safety of the public.

Should you have any questions regarding this matter, please contact Mr. Sherwood R. Zimmerman at (919) 836-6242.

Yours very truly,

A. B. Cutter - Vice President
Nuclear Engineering & Licensing

ABC/HS/ccc (1181NLU)

cc: Mr. L. W. Garner (NRC-BNP)
Dr. J. Nelson Grace (NRC-RII)
Mr. M. Grotenhuis (NRC)