



Entergy Operations, Inc.
1448 S.R. 333
Russellville, AR 72901
Tel 501 858-5000

10CFR50.46(a)(3)(ii)

May 29, 1996

OCAN059608

U. S. Nuclear Regulatory Commission
Document Control Desk
Mail Station P1-137
Washington, DC 20555

Subject: Arkansas Nuclear One - Units 1 and 2
Docket Nos. 50-313 and 50-368
License Nos. DPR-51 and NPF-6
Errors or Changes in the Emergency Core Cooling
System Evaluation; Annual Report For 1995

Gentlemen:

10CFR50.46(a)(3)(ii) requires licensees to report each change to or error discovered in an acceptable evaluation model or in the application of such model for the emergency core cooling system (ECCS) at least annually and the estimated effect these changes or errors have on the limiting ECCS analysis. The purpose of this report is to provide the required information for Arkansas Nuclear One (ANO). Due to a delay in the receipt of the report from the vendor, the information for ANO-1 will be provided in the near future.

No significant changes or errors, i.e., peak cladding temperature (PCT) change of greater than 50 degrees F, have been identified for the ABB-Combustion Engineering evaluation model used for ANO-2. One minor error in the input processing for the COMFERC-II refill/reflood code for the large break loss of coolant accident (LBLOCA) analysis was found and corrected. This had no effect on the PCT for the LBLOCA analysis. The sum of the absolute magnitudes of the PCT changes for the LBLOCA analysis from all reports to date continues to be less than 1 degree F. No change occurred in the PCT for the small break LOCA or post-LOCA long term cooling. The attached report (CENPD-279, Supplement 7) is the Annual Report on the ABB-Combustion Engineering ECCS Performance Evaluation Models for 1995 and provides a detailed discussion of these errors and associated resolutions.

There was one change to the ANO-2 specific inputs to the LBLOCA evaluation model. This change was made during the assessment of the cycle 12 physics data. The linear heat rate was increased from 12.8 kW/ft to 13.5 kW/ft. This is a change in the input data utilized in the analysis and not a change to or error in the model or in the application of the model.

9606070099 960529
PDR ADOCK 05000313
R PDR

010004

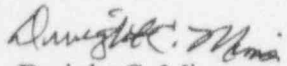
A001
11

U. S. NRC
May 29, 1996
OCAN059608 Page 2

Therefore, the requirements of 10CFR50.46 are not applicable, and this input change is being submitted for information only.

Should you have any questions, please contact me.

Very truly yours,


Dwight C. Mims
Director, Nuclear Safety

DCM/nbm
Attachment

cc: Mr. Leonard J. Callan
Regional Administrator
U. S. Nuclear Regulatory Commission
Region IV
611 Ryan Plaza Drive, Suite 400
Arlington, TX 76011-8064

NRC Senior Resident Inspector
Arkansas Nuclear One
P.O. Box 310
London, AR 72847

Mr. George Kalman
NRR Project Manager Region IV/ANO-1 & 2
U. S. Nuclear Regulatory Commission
NRR Mail Stop 13-H-3
One White Flint North
11555 Rockville Pike
Rockville, MD 20852