

STATE OF ILLINOIS

DEPARTMENT OF NUCLEAR SAFETY

1035 OUTER PARK DRIVE SPRINGFIELD 62704 (217) 546-8100

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March 19, 1985

Mr. Leonard N. Olshan, Project Manager Byron Station Units 1 & 2 Division of Licensing Office of Nuclear Reactor Regulation U.S. Nuclear Regulatory Commission Washington, D.C. 20555

SUBJECT: Comments of Supplement 6 to NUREG-0876, Byron Station Units 1 & 2 Safety Evaluation Report

In Reply Please Refer To: DOE-85-038

Dear Mr. Olshan:

After a review of subject document, the following comments are offered for your consideration:

- 3 design criteria for structures, systems, & components
 - 3.6 Protection Against Effects Associated with the Postulated Rupture of Piping
 - 3.6.2 Determination of break locations and dynamic effects associated with the postulated rupture of piping.

Comment: NUREG/CR 2913, "Two Phase Jet Loads" was used in Byron design verification, although it is not currently approved for general use and is under review by NRC.

NRC states that NUREG/CR 2913 use was an acceptable alternative provided the separation distance is at least 8 pipe diameters (source pipe size).

IDNS questions the use of NUREG/CR 2913 since it is not currently approved by NRC. Although it was not used directly for design purposes, rather design verification, it would appear unacceptable even for that purpose.

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- 2) 4 Reactor
 - 4.4 Thermal & Hydraulic Design
 - 4.4.1 Departure from nucleate boiling methodology
- <u>Comment:</u> The method of detecting venturi fouling, "by reaching through the inspection port & touching the venturi surface", appears to be a too subjective method and in relation to the visual methods proposed should be expanded to be more definitive as to acceptance criteria.
- 3) 6 Engineered Safety Features
 - 6.5 Fission Production Removal & Control System
 - 6.5.1 Engineered Safety Features Atmospheric Cleanup System
- <u>Comment:</u> There should be a better explanation regarding the three ventilation systems which traverse the upper cable spreading room not affecting the control room.

Thank you for the opportunity to review the Byron Unit 1 & 2 safety evaluation report. Your consideration of the above comments are appreciated.

Sincedely

Michael C. Parker, Chief Division of Engineering

MCP: RRM: bkw

cc: T. Lash

G. Wright

J. Keppler, NRC Region III

R. Lickus, NRC Region III