NRC Form 366 (9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
VERMONT YANKEE NUCLEAR POWER STATION	0 5 0 0 0 2 7 1	1 OF 0 3

TITLE (4)
1984 Appendix J-Type B and C Testing

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
0 6	1 6	8 4	8 4	— 0 1 1	— 0 2	0 3	0 8	8 5		0 5 0 0 0
										0 5 0 0 0

OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)			
N		20.402(b)	20.405(c)	50.73(a)(2)(iv)	73.71(b)
POWER LEVEL (10)		20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
0 0 0		20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	X OTHER (Specify in Abstract below and in Text, NRC Form 366A)
		20.405(a)(1)(iii)	X 50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
		20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	10CFR50 App. J.
		20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
James P. Pelletier, Plant Manager	AREA CODE 8 0 2 — 2 5 7 - 7 7 1 1

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	
X	S B	V	R 3 4 0	Y		X	B B	V	A 1 8 0	Y	
X	A A	V	L 6 3 1	Y		B	S J	V	A 3 9 1	Y	

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

While performing Type C Leak Rate Testing, MSIV-86B, CRD-412A, PCAC-V16-19-8, FDW-96A and CA-89C were found to have seat leakage above that permitted by Tech. Spec. section 3.7.A.4. This resulted in the total Appendix J Type B and C limit of 14.75 lbm/Hr. being exceeded which does not meet Tech. Spec. section 3.7.A.3 requirements. (By procedure, VY uses the maximum pathway leakage in calculating total penetration leakage.)

Vermont Yankee has performed maintenance on all of the above valves and retested them to ensure that both total penetration and individual valve seat leakages are within Tech. Specs.

NRC Form 366A (9-83)

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
VERMONT YANKEE NUCLEAR POWER STATION	0 5 0 0 0 2 7 1	8 4	— 0 1 1	— 0 2	0 2 OF 0 3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

While performing Type C Leak Rate Testing MSIV-86B, CRD-412A, PCAC V16-19-8, FDW-96A and CA-89C were found to have seat leakage above that permitted by Tech. Spec. section 3.7.A.4. This resulted in the total Appendix J Type B and C limit of 14.75 lbm/Hr being exceeded which does not meet Tech. Spec. section 3.7.A.3 requirements. (Note: By procedure, VY uses the maximum pathway leakage in calculating total penetration leakage.)

For MSIV-86B, CRD-412A, PCAC V16-19-8 and CA-89C, a second isolation valve in the applicable system was tested and met the acceptable criteria. For FDW-96A, containment was provided by the water seal on the inboard check valve and the plant's capability to maintain a pressure greater than P_a on the feedwater system.

Total penetration leakage based on the minimum pathway leakage equaled 9.83 lbm/Hr. Minimum pathway leakage is used to determine Appendix J Type A total containment leakage.

Vermont Yankee has performed maintenance on all of the above valves. For CRD-412A and CA-89C, maintenance involved disassembling the valve and cleaning the seating surface. For MSIV 86B, maintenance involved lapping the seating surface and replacing the pilot valve disc. For PCAC V16-19-8 the seats were replaced. No similar events have been reported on the above valves in the last five years. Based on this the above events are not considered significant repeat failures.

For FDW-96A, maintenance involved replacing Steven's soft seats with Stillman soft seats. The Steven's seats were scheduled to be replaced due to a known manufacturing defect. A similar event was reported on FDW 96A as LER 83-10. Vermont Yankee retested the above valves to ensure that both total penetration and individual valve seat leakages are within Tech. Specs.

Based on the above there were no adverse consequences to the health and safety of the public.

LER 84-11

NRC Form 366B
(9-83)

LICENSEE EVENT REPORT (LER) FAILURE CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150–0104
EXPIRES: 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6) YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	PAGE (3)
Vermont Yankee	0 5 0 0 0 2 7 1	8 4	0 1 1	0 2	0 3 OF 0 3

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFAC-TURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFAC-TURER	REPORTABLE TO NPRDS
X	LE	V	C339						

NRC FORM 366B
(9-83)

VERMONT YANKEE NUCLEAR POWER CORPORATION

P. O. BOX 157
GOVERNOR HUNT ROAD
VERNON, VERMONT 05354

March 8, 1985

VYV85-146

U.S. Nuclear Regulatory Commission
Document No. 50-271
Washington, D.C. 20555

REFERENCE: Operating License DPR-28
Docket No. 50-271
Reportable Occurrence No. LER 84-11, Revision 2

Dear Sirs:

As defined by 10CFR50.73, we are reporting the attached Reportable Occurrence as LER 84-11, Revision 2.

Very truly yours,

VERMONT YANKEE NUCLEAR POWER CORPORATION

James P. Pelletier

James P. Pelletier
Plant Manager

RDP/jbb

cc: Regional Administrator
USNRC Office of Inspection and Enforcement
Region I
631 Park Avenue
King of Prussia, Pennsylvania 19406

IE22
1/1