

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Monticello	DOCKET NUMBER (2) 0 5 0 0 0 2 6 1 3	PAGE (3) 1 OF 0 2
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TITLE (4)  
Trip of Shutdown Cooling During Primary Containment ILRT

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)							
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)					
1	2	07	8	4	8	4	0	3	4	0	0	0	5	0	0	0
1	2	07	8	4	8	4	0	3	4	0	0	0	5	0	0	0

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 8: (Check one or more of the following) (11)

OPERATING MODE (9) N	20.402(b)	<input checked="" type="checkbox"/>	50.73(a)(2)(iv)	<input type="checkbox"/>	73.71(b)
	20.408(a)(1)(i)	<input type="checkbox"/>	50.73(a)(2)(v)	<input type="checkbox"/>	73.71(c)
	20.408(a)(1)(ii)	<input type="checkbox"/>	50.73(a)(2)(vii)	<input type="checkbox"/>	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
	20.408(a)(1)(iii)	<input type="checkbox"/>	50.73(a)(2)(viii)(A)	<input type="checkbox"/>	
	20.408(a)(1)(iv)	<input type="checkbox"/>	50.73(a)(2)(viii)(B)	<input type="checkbox"/>	
	20.408(a)(1)(v)	<input type="checkbox"/>	50.73(a)(2)(ix)	<input type="checkbox"/>	

LICENSEE CONTACT FOR THIS LER (12)

NAME Jeff Nelson, Production Engineer	TELEPHONE NUMBER
	AREA CODE: 6 1 2      2 9 5 - 5 1 5 1

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE)       NO

EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

During pressurization of primary containment for the Intergrated Leak Rate Test (ILRT), the inboard and outboard isolation valves for the RHR shutdown cooling supply line closed due to a trip of the RHR suction pressure interlock. This action isolated the suction for the RHR pump in service and caused an automatic trip of the pump.

The interlock was bypassed to allow shutdown cooling to be put back in service. The procedure for performing the ILRT will be revised to prevent reoccurrence of this event.

JEFF  
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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1)  Monticello	DOCKET NUMBER (2)  0 5 0 0 0 2 6 3 8 4	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
		8 4	0 3 4	0 0 0	2	0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On December 7, 1984 at 8:09 C.D.T. during pressurization of primary containment for the Integrated Leak Rate Test (ILRT) the #11 RHR (BO) pump (P) tripped while in service in shutdown cooling. The plant was in a maintenance/refueling outage at the time.

The pump trip was caused by closure of the shutdown cooling supply line isolation valves (ISV), MO-2029 and MO-2030. The RHR pumps are tripped automatically when the isolation valves are closed by a loss of suction interlock. The isolation closure was initiated by the RHR suction pressure interlock (IEL) which protects the RHR suction piping from over pressurization. The setpoint for this interlock is 75 psig at the RHR pumps suction. Due to static head this corresponds to a reactor pressure of approximately 40 psig. The containment pressure of 40 psig pressurized the reactor through the reactor (RCT) head vent line and tripped the RHR suction pressure interlock.

The interlock setpoint had recently been reduced by 18 psi to compensate for the elevation difference between the isolation valves and the suction of the RHR pumps. The ILRT procedure did not provide for bypassing the interlock.

The RHR suction pressure interlock was bypassed to allow the RHR system to be put back in operation. The bypass was left in place for completion of the ILRT and then was removed. The bypass was determined to be acceptable because isolation function operability was not required by Technical Specifications and the final containment pressure of 43 psig would not damage the RHR suction piping. The ILRT procedure will be revised to prevent a recurrence of this event.

The RHR system was isolated for approximately 1 hour. This had no significant effect on vessel or core temperature. The ability of the RHR system to provide the Low Pressure Coolant Injection (LPCI) function was not impaired. All systems functioned as designed. This occurrence had no effect on the health and safety of the public. No other similar previous reportable events have occurred.



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U S Nuclear Regulatory Commission  
Document Control Desk  
Washington, DC 20555

MONTICELLO NUCLEAR GENERATING PLANT  
Docket No. 50-263 License No. DPR-22

Trip of Shutdown Cooling During Primary Containment ILRT

The License Event Report for this occurrence is attached.

This event was reported via Emergency Notification System per 10 CFR Part 72 on December 7, 1984.

*Thonica Vik*

*for* David Musolf  
Manager - Nuclear Support Services

DMM/MMV/dab

c: Regional Administrator-III, NRC  
NRR Project Manager, NRC  
Resident Inspector, NRC  
MPCA  
Attn: J W Ferman

Attachment

IE22  
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