LASALLE NUCLEAR POWER STATION

UNIT 1

MONTHLY PERFORMANCE REPORT

JANUARY 1985

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-373

LICENSE NO. NPF-11

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I. INTRODUCTION

The LaSalle Nuclear Power Station is a Two Unit Facility Located in Marseilles, Illinois. Each Unit is a Boiling Water Reactor with a designed electrical output of 1078 MWe net. The Station is owned by Commonwealth Edison Company. The Architect/Engineer was Sargent & Lundy, and the primary construction contractor was Commonwealth Edison Company.

The condenser cooling method is a closed cycle cooling pond. Unit One is subject to License Number NPF-11, issued on April 17, 1982. The date of initial criticality was June 21, 1982. Unit Two is subject to license number NPF-18, issued on December 16, 1983. The date of initial criticality was March 10, 1984.

This report was compiled by Randy S. Dus telephone number (815)357-6761, extension 324.

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II. MONTHLY REPORT FOR UNIT ONE

A. SUMMARY OF OPERATING EXPERIENCE FOR UNIT ONE

JANUARY 1-5

Jan. 1, 0001 hours - Reactor power at 90%.
Jan. 5, 1310 hours - Reactor manually scrammed due to a generator field ground. The reactor was critical for 109 hours and 10 minutes.

JANUARY 6-31

Jan. 6, 1840 hours - Reactor critical.
Jan. 7, 0615 hours - Generator synchronized to grid.
Jan. 7, 0700 hours - Reactor power at 36%.
Jan. 8, 0700 hours - Reactor power at 71%.
Jan. 11, 1500 hours - Reactor power at 94%.
Jan. 13, 0145 hours - Reactor power at 85%.
Jan. 26, 0700 hours - Reactor power was reduced to 34% for a rod sequence change.
Jan. 26, 1035 hours - The main turbine was manually tripped for maintenance work on the #3 turbine control valve.
Jan. 26, 1803 hours - Generator synchronized to grid.
Jan. 27, 0820 hours - Reactor power raised to 58%.
Jan. 27, 2800 hours - Reactor power at 72%.
Jan. 30, 1600 hours - Reactor power at 90%. The reactor was critical for 714 hours and 30 minutes.

B. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY RELATED MAINTENANCE.

1. Amendments to facility license or Technical Specification.

   Amendment No. 20-This amendment eliminates the Reactor Water Cleanup pump room ambient and differential temperature monitoring requirments.

2. Facility or procedure changes requiring NRC approval.

   There were no facility or procedure changes requiring NRC approval.

3. Tests and Experiments requiring NRC approval.

   There were no tests or experiments requiring NRC approval.

4. Corrective maintenance of safety related equipment.

   The following table (Table 1) presents a summary of safety-related maintenance completed on Unit One during the reporting period. The headings indicated in this summary include: Work Request numbers, LER numbers, Component Name, Cause of Malfunction, Results and Effects on Safe Operation, and Corrective Action.

TABLE 1

CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

WORK REQUEST	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS	CORRECTIVE ACTION ON SAFE OPERATION
L35771	Drywell penetration No. E-32.	Penetration leaks excessively.	Penetrations required to be fully charged.	Recharged penetration
L39371	RHR heat exchanger recalibrations.	Recalibrate associated instrumentation.	Instruments required to be calibrated periodically.	Calibrated instrumentation.
L43602	1E12-F009 Shutdown Cooling suction isolation valve	Overload reset pushbutton does not contact OL reset due to misalingnment.	Not able to reset thermal overloads	Remounted reset pushbutton.
L44022	VC/VE ammonia detector	Chemcassette tape transport carriage mechanism beginning to corrode.	Preventative maintenance	Replaced mechanism
L44787	VE return fan isolation damper	Damper would not open, preventing the return fan from running.	Inadequate Aux. Elec. Equip. room ventilation.	Exercised sticking limit switch and cycled damper.
L44892	Div I Post LOCA O_2 monitor	Recorder failed downscale.	Div. II still operable.	Recalibrated per LIS-PC-07.
L44982	"B" APRM 5 volt power supply.	Power supply failed	No APRM indication from channel "B".	Replaced power supply
L45019	A Train "B" ammonia detector.	Ammonia detector failed to trip during LIS-VC-053.	Required to be operable.	Replaced master fault lamp and flow lamp & cleaned optics.
L45062	"F" APRM 5 volt power supply.	Power supply failed	No APRM indication from channel "F".	Replaced power supply.
L45107	PCIS Level switch	Switch out of calibration	Actuates earlier than calibration sheets dictate	Performed applicable portions of LIS-NB-102

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TABLE 1

CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

WORK REQUEST	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS	CORRECTIVE ACTION ON SAFE OPERATION
L45112	#2 Control Valve	Limit Switch actuating ARM missing.	Could not get half-scram when CV#2 closed during LOS-RP-M4.	Replaced missing limit switch actuating apm.
L45363	Suppression Pool Level recorder	Control Room pool level indicating lower than local sight glass.	Inaccurate suppression pool level indication.	Recalibrated Loop Per LIS-CM-01.
L45603	SBGT WRGM	SBGT WRGM Inoperable.	Hi range of SBGT WRGM has erratic signal.	Cleaned power supply

C. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for LaSalle Nuclear Power Station, Unit One, occurring during the reporting period, January 1 through January 31, 1985. This information is provided pursuant to the reportable occurrence reporting requirements as set forth in 10CFR 50.73.

Licensee Event Report Number	Date	Title of Occurrence
84-086-00	9/27/84	Radwaste Discharge With Incorrect Monitor Setpoint.
84-087-00	12/26/84	Suppression Pool High Level.
84-088-00	12/2/84	Potential Failure of Containment Penetration.
84-089-00	01/09/85	Actuation of VC Ammonia Detector.
84-090-00	01/09/85	HPCS Transfer to Suppression Pool on HI Suppression Pool Level.
84-091-00	1/11/85	Actuation of B VC Ammonia Detector.
84-092-00	12/30/84	Unsecured Hi-Rad Door.
84-093-00	12/19/84	Unsecured Hi-Rad Area.
85-001-00	1/2/85	Reactor Vessel Hi-Level HPCS Injection Valve Closure Switch Out of Cal.
85-002-00	1/5/85	Reactor Scram on Low Level.
85-003-00	1/6/85	Reactor Water Cleanup Isolation.

D. DATA TABULATIONS

The following data tabulations are presented in this report:

1. Operating Data Report

2. Average Daily Unit Power Level

3. Unit Shutdowns and Power Reductions

1. OPERATING DATA REPORT

DOCKET NO. 050-373
UNIT LaSalle One
DATE February 10, 1985
COMPLETED BY Randy S. Dus
TELEPHONE (815)357-6761

OPERATING STATUS

1. REPORTING PERIOD: January 1985 GROSS HOURS IN REPORTING PERIOD: 744
2. CURRENTLY AUTHORIZED POWER LEVEL (MWt):3323 MAX DEPEND CAPACITY (MWe-Net): 1036 DESIGN ELECTRICAL RATING (MWe-Net):1078
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A
4. REASONS FOR RESTRICTION (IF ANY):

		THIS MONTH	YR TO DATE	CUMULATIVE
5	NUMBER OF HOURS REACTOR WAS CRITICAL	714.5	714.5	6995
6.	REACTOR RESERVE SHUTDOWN HOURS	29.5	29.5	1195
7.	HOURS GENERATOR ON LINE	695.5	695.5	6751
8.	UNIT RESERVE SHUTDOWN HOURS	0.0	0.0	0.0
9.	GROSS THERMAL ENERGY GENERATED (MWH)	1884113	1884113	18707402
10.	GROSS ELEC. ENERGY GENERATED (MWH)	618288	618288	6088931
11.	NET ELEC. ENERGY GENERATED (MWH)	594395	594395	5789457
12.	REACTOR SERVICE FACTOR	96.0%	96.0%	73.4%
13.	REACTOR AVAILABILITY FACTOR	100%	100%	86.0%
14.	UNIT SERVICE FACTOR	93.5%	93.5%	70.9%
15.	UNIT AVAILABILITY FACTOR	93.5%	93.5%	70.9%
16.	UNIT CAPACITY FACTOR (USING MDC)	77.1%	77.1%	58.7%
17.	UNIT CAPACITY FACTOR(USING DESIGN MWe)	74.1%	74.1%	56.4%
18.	UNIT FORCED OUTAGE RATE	6.5%	6.5%	14.3%

19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH)
20. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: N/A

2. AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO: 050-373
UNIT: LASALLE ONE
DATE: FEBRUARY 10, 1985
COMPLETED BY: Randy S. Dus
TELEPHONE: (815) 357-6761

MONTH: JANUARY, 1985

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1.	1030	17.	920
2.	1030	18.	912
3	972	19.	916
4.	1026	20.	918
5.	588	21.	915
6.	0	22.	915
7.	182	23.	917
8.	709	24.	916
9.	641	25.	870
10.	788	26.	233
11.	979	27.	621
12.	940	28.	611
13.	914	29.	779
14.	849	30.	944
15.	895	31.	944
16.	914		

ATTACHMENT E

3. UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 050-374
UNIT NAME LaSalle One
DATE February 10, 1985
COMPLETED BY Randy S. Dus
TELEPHONE (815)357-6761

REPORT MONTH JANUARY 1985

NO.	DATE	TYPE F: FORCED S: SCHEDULED	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS
1	85/01/5	F	41.1	A	2	While the unit was at approximately 99% power, a generator field ground occurred due to high stator cooling water conductivity.
2	85/01/26	F	7.5	A	1	The main turbine was tripped so that maintenance could be performed on the #3 control valve.

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2. ECCS Systems Outages

The following outages were taken on ECCS Systems during the reporting period.

OUTAGE NO.	EQUIPMENT	PURPOSE OF OUTAGE
1-3-85	1E12-C300D	Megger Motor
1-4-85	1E12-C300A	Megger Motor
1-6-85	1E12-F068A	Repair Valve
1-15-85	1E22-C302B	Overhaul Diesel
1-16-85	1E22-C302B	Overhaul Diesel
1-15-85	1E12-C300A	Megger Motor
1-41-85	1A D/G	Lubrication
1-42-85	1A D/G	Lubrication
1-44-85	1E12-F009	Repair Breaker
1-48-85	RHR Water Leg Pump	Lubrication
1-49-85	"B" RHR Service Water Pump	LEP-GM-120
1-52-85	"A" RHR Pump	Lubrication

3. Off-Site Dose Calculation Manual

There were no changes to the off-site dose calculations manual during this reporting period.

4. Radioactive Waste Treatment Systems.

There were no changes made to the radioactive waste treatment system during this reporting period.

E. UNIQUE REPORTING REQUIREMENTS

1. Safety/Relief valve operations for Unit One.

DATE	VALVES ACTUATED	NO & TYPE ACTUATION	PLANT CONDITION	DESCRIPTION OF EVENT

There were no safety relief valve actuations for this reporting period.

LASALLE NUCLEAR POWER STATION

UNIT 2

MONTHLY PERFORMANCE REPORT

JANUARY 1985

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-374

LICENSE NO. NPF-18

DOCUMENT ID 0036r/0005r

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I. INTRODUCTION

The LaSalle Nuclear Power Station is a Two Unit Facility Located in Marseilles, Illinois. Each Unit is a Boiling Water Reactor with a designed electrical output of 1078 MWe net. The Station is owned by Commonwealth Edison Company. The Architect/Engineer was Sargent & Lundy, and the primary construction contractor was Commonwealth Edison Company.

The condenser cooling method is a closed cycle cooling pond. Unit One is subject to License Number NPF-11, issued on April 17, 1982. The unit commenced commercial generation of power on January 1, 1984. Unit Two is subject to license number NPF-18, issued on December 16, 1983. The date of initial criticality was March 10, 1984.

This report was compiled by Randy S. Dus, telephone number (815)357-6761, extension 324.

II. MONTHLY REPORT FOR UNIT TWO

A. SUMMARY OF OPERATING EXPERIENCE FOR UNIT TWO

JANUARY 1-31

Jan. 1, 0001 hours - The unit started the reporting period at 90% power.
Jan. 4, 0700 hours - Reactor power at 99%.
Jan. 4, 2200 hours - Reducing Reactor power for a Rod Sequence exchange.
Jan. 5, 0700 hours - Reactor power at 70%.
Jan. 6, 0700 hours - Reactor power at 82%.
Jan. 9, 1400 hours - Reactor power at 97%.
Jan. 21, 1920 hours - Reactor power reduced to 75% due to stuck RBCCW control valve.
Jan. 22, 2300 hours - Reactor power at 96%. The reactor was critical for 744 hours.

B. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY RELATED MAINTENANCE.

1. Amendments to facility license or Technical Specifications.

   Amendment No. 7 - This amendment eliminates the Reactor Water Cleanup pump room ambient and differential temperature monitoring requirements.

2. Facility or procedure changes requiring NRC approval.

   There were no facility or procedure changes requiring NRC approval during the reporting period.

3. Tests and experiments requiring NRC approval.

   There were no tests or experiments requiring NRC approval during the reporting period.

4. Corrective Maintenance of Safety Related Equipment.

   The following table (Table 1) presents a summary of safety-related maintenance completed on Unit One during the reporting period. The headings indicated in this summary include: Work Request numbers, LER Numbers, Component Name, cause of malfunction, results and effects on safe operation, and corrective action.

TABLE 1

CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

WORK REQUEST	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS	CORRECTIVE ACTION ON SAFE OPERATION
L35213	RCIC pump suction pressure gauge.	Pressure gauge giving false indication when pump is not running	Gauge is reading approximately 50 psig low.	Recalibratd Gage.
L36890	"2B" Diesel Generator	Remove and renew the lube oil & lube oil filter.	The lube oil has been diluted with fuel oil.	Changed oil filter and oil.
L39972	VG Filter train	High DP across demister causing control room alarms.	Actual measured DP is satisfactory. Improper flow elements installed.	Installed new probes and recalibrated.
L41953	C RHR pump	Inspect pump/driver coupling to be sure that set screws are in place.	Preventive maintenance	Removed coupling spacer and exposed nut & set screws. Found to be tight.
L44543	B RHR service water PRM.	Hi back ground in area of PRM.	Unable to get true service water indication.	Added more lead blankets around PRM.
L44856	Div. II Post Loca Monitor	Reading zero locally and 2% in control room.	Oxygen concentration required to be known at all times.	Replaced O_2 analyzer amp card and recalibrated.
L44907	24 point cam/2PL15J	Sample pump cycles on and off every 30 seconds	Continuous air monitor required to be sampling at all times.	Tightened vacuum leaks at flow indicator pipe connections. Recalibrated Hi-Lo trip switches.
L45024	LPRM 24-41	LPRM drifting low	Showing downscale reading.	New current gain applied to detector channel.

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CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

WORK REQUEST	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS	CORRECTIVE ACTION ON SAFE OPERATION
L45185	Suppression Pool Level recorder.	High indicated level compared to local sight glass and other instruments.	Inaccurate pool level indication	Backfilled reference leg.
L45235	HPCS D/G	Will not start due to low oil pressure.	Required to be operable.	Found oil filter installed backwards.
L45314	IRM/APRM flux recorder	Pen is very slow to respond	Untrue flux indications.	Tightened carriage assembly, cleaned and lubricated gears.
L45383	Excess Flow Check Vlv	Check valve is stuck	Results in a incorrect suppression pool level.	Disassembled valve, cleaned and re-assembled.

C. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for LaSalle Nuclear Power Station, Unit Two, occurring during the reporting period, January 1 through January 31, 1985. This information is provided pursuant to the reportable occurrence reporting requirements as set forth in 10CFR 50.73.

Licensee Event Report Number	Date	Title of Occurrence
84-082-00	12/7/84	Uncontrolled Access to High Radiation Area.
84-083-00	12/8/84	Primary Containment Vacuum Breaker Cycled.
84-084-00	12/14/84	Failure of RCIC high Temperature Isolation Switch.
84-085-00	12/14/84	Scram from Hi Vibration Turbine Trip.
84-086-00	1/8/85	Strut #RH14-2820X Inoperable.
84-087-00	1/7/85	High Suppression Pool Level-HPCS Suction Swap.
84-088-00	1/9/85	Vacuum Breaker a Cycled.
84-089-00	12/15/84	Reactor Water Cleanup Hi Delta Flow Isolation.
84-090-00	12/16/84	Vacuum Breaker Cycling.
84-091-00	12/17/84	Vacuum Breaker Actuation.
84-092-00	12/17/84	Vacuum Breaker Cycled.
84-093-00	12/20/84	Reactor Water Cleanup Hi Delta Flow Isolation.
85-001-00	1/7/85	HPCS Suction Valves Swapped.

D. DATA TABULATIONS

The following data tabulations are presented in this report:

1. Operating Data Report

2. Average Daily Unit Power Level

3. Unit Shutdowns and Power Reductions

1. OPERATING DATA REPORT

DOCKET NO. 050-374
UNIT LaSalle Two
DATE February 10, 1985
COMPLETED BY Randy S. Dus
TELEPHONE (815)357-6761

OPERATING STATUS

1. REPORTING PERIOD: January 1985 GROSS HOURS IN REPORTING PERIOD: 744
2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 3323 MAX DEPEND CAPACITY (MWe-Net): 1036 DESIGN ELECTRICAL RATING (MWe-Net): 1078
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A
4. REASONS FOR RESTRICTION (IF ANY):

		THIS MONTH	YR TO DATE	CUMULATIVE
5	NUMBER OF HOURS REACTOR WAS CRITICAL	744	744	2355.8
6.	REACTOR RESERVE SHUTDOWN HOURS	0.0	0.0	125.3
7.	HOURS GENERATOR ON LINE	744	744	2281.4
8.	UNIT RESERVE SHUTDOWN HOURS	0.0	0.0	0.0
9.	GROSS THERMAL ENERGY GENERATED (MWH)	2362207	2362207	6874799
10.	GROSS ELEC. ENERGY GENERATED (MWH)	785448	785448	2270443
11.	NET ELEC. ENERGY GENERATED (MWH)	759220	759220	2151337
12.	REACTOR SERVICE FACTOR	100%	100%	93.5%
13.	REACTOR AVAILABILITY FACTOR	100%	100%	98.5%
14.	UNIT SERVICE FACTOR	100%	100%	90.5%
15.	UNIT AVAILABILITY FACTOR	100%	100%	90.5%
16.	UNIT CAPACITY FACTOR (USING MDC)	98.5%	98.5%	82.4%
17.	UNIT CAPACITY FACTOR(USING DESIGN MWe)	94.7%	94.7%	79.2%
18.	UNIT FORCED OUTAGE RATE	0%	0%	9.5%

19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH): There is an outage scheduled to begin on March 1, 1985 for maintenance and surveillances. This outage is expected to last approximately nine weeks.
20. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP N/A

2. AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO: 050-374
UNIT: LASALLE TWO
DATE: February 10,1985
COMPLETED BY: Randy S. Dus
TELEPHONE: (815) 357-6761
MONTH: January 1985

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1.	1032	17.	1066
2.	1038	18.	1054
3	1026	19.	1062
4.	1046	20.	1063
5.	702	21.	1015
6.	747	22.	971
7.	904	23.	1061
8.	1048	24.	1062
9.	1042	25.	1060
10.	1054	26.	1067
11.	1038	27.	1062
12.	1037	28.	1064
13.	1041	29.	1062
14.	1000	30.	1059
15.	1035	31.	1060
16.	1060		

ATTACHMENT E

3. UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 050-374
UNIT NAME LaSalle Two
DATE FEBRUARY 10, 1985
COMPLETED BY Randy S. Dus
TELEPHONE (815)357-6761

REPORT MONTH JANUARY 1985

NO.	DATE	TYPE F: FORCED S: SCHEDULED	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS
1	85/01/05	S	0.0	H	5	Power reduction for rod sequence exchange.

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E. UNIQUE REPORTING REQUIREMENTS

1. Safety/Relief Valve Operations for Unit Two.

DATE	VALVES ACTUATED	NO & TYPE ACTUATIONS	PLANT CONDITION	DESCRIPTION OF EVENT

There were no safety relief valve actuators during this reporting period.

2. ECCS Systems Outages

The following outages were taken on ECCS Systems during the reporting period.

OUTAGE NO.	EQUIPMENT	PURPOSE OF OUTAGE
2-1-85	LPCS pump	Lubrication
2-6-85	B RHR pump	Lubrication
2-13-85	2A D/G Air Compressor	Replace blown Head Gasket
2-14-85	2A D/G Air compressor	Replace blown Head Gasket
2-20-85	2B D/G	LIP-DG-03
2-21-85	2B D/G Air compressors	Recalibrate instruments
2-40-85	2B RHR pump	Oil samples
2-51-85	A RHR pump	Inspect coupling
2-57-85	B RHR pump	Inspect coupling
2-61-85	2A RHR Service Water pump	Lubrication
2-62-85	2A RHR service Water pump	Megger motor
2-65-85	"C" RHR pump	Coupling Check
2-75-85	"B" RHR Service Water pump	Polarization check
2-77-85	"C" RHR pump suction vent	Remove/Replace Sight Glass

3. Off-Site Dose Calculation Manual

There were no changes to the off-site dose calculations manual during this reporting period.

4. Radioactive Waste Treatment Systems.

There were no changes made to the radioactive waste treatment system during this reporting period.

Commonwealth Edison
LaSalle County Nuclear Station
Rural Route #1, Box 220
Marseilles, Illinois 61341
Telephone 815/357-6761

February 10, 1985

Director, Office of Management Information
and Program Control
United States Nuclear Regulatory Commission
Washington, D.C. 20555

ATTN: Document Control Desk

Gentlemen:

Enclosed for your information is the monthly performance report covering LaSalle County Nuclear Power Station for the period covering January 1 through January 31, 1985.

Very truly yours,

C E Sargent

for G. J. Diederich
Superintendent
LaSalle County Station

GJD/RSD/crh

Enclosure

xc: J. G. Keppler, NRC, Region III
NRC Resident Inspector LaSalle
Gary Wright, Ill. Dept. of Nuclear Safety
D. P. Galle, CECo
D. L. Farrar, CECo
INPO Records Center
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