

LICENSEE EVENT REPORT (LER)

APPROVED OMB NO. 3160-0108
EXPIRES - 9/31/83

FACILITY NAME (1) **Limerick Generating Station - Unit 1** DOCKET NUMBER (2) **0 5 0 0 0 3 1 5 1 2 1** OF **0 1 3**

TITLE (4) **Reactor Enclosure Penetration Seals**

EVENT DATE (8)			LER NUMBER (8)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER (2)
11	29	84	84	022	do	12	28	84			0 5 0 0 0 1 1 1
<small>THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §. (Check one or more of the following) (11)</small>											

OPERATING MODE (8) 5	20.402(a)	20.406(a)	60.73(a)(2)(iv)	73.31(a)
POWER LEVEL (16) 0, 0, 0	20.406(a)(1)(ii)	60.36(a)(1)	60.73(a)(2)(v)	73.31(a)
	20.406(a)(1)(iii)	60.34(a)(2)	60.73(a)(2)(vi)	OTHER (Specify in Abstract below and in Text, NRC Form 306A)
	20.406(a)(1)(iv)	X 60.73(a)(2)(ii)	60.73(a)(2)(vii)(A)	
	20.406(a)(1)(v)	60.73(a)(2)(iii)	60.73(a)(2)(viii)(B)	
	20.406(a)(1)(vi)	60.73(a)(2)(iv)	60.73(a)(2)(ix)	

LICENSEE CONTACT FOR THIS LER (12) NAME **B. L. Clark, Senior Engineer-Special Projects** TELEPHONE NUMBER **2 1 5 8 4 1 - 5 0 1 4 7**

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC
B	K, P			N					

SUPPLEMENTAL REPORT EXPECTED (14) YES (If yes, complete EXPECTED SUBMISSION DATE) NO

EXPECTED SUBMISSION DATE (15) MONTH **11** DAY **11** YEAR **84**

ABSTRACT (Limit to 1400 words, i.e., approximately fifteen single-space typewritten lines) (16)

Abstract: 84-022

During fire damper inspection on November 29, 1984, two reactor enclosure fire penetrations were discovered not sealed. The missing seals were identified as a seismic gap fire seal at a fire damper at elevation 201 ft. and a spare conduit penetration in elevation 253 feet. Fire watches/patrols were established immediately in the affected areas. Additional inspection identified a fire penetration seal missing on elevation 352 feet.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1) Limerick Generating Station Unit 1	DOCKET NUMBER (2) 0500035284	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		84	022	00	02	OF	03

TEXT (if more space is required, use additional NRC Form 366a (1))

Description of the Event:

During fire damper inspection on November 29, 1984, prior to initial criticality, two reactor enclosure fire penetrations were discovered not sealed. The missing seals were identified as a seismic gap fire seal at a fire damper at elevation 201 feet between the reactor core isolation cooling (RCIC) system pipe chase and RCIC access hatch area; and a spare conduit penetration on elevation 253 feet near the control rod drive (CRD) system access hatch where test cables for the power ascension program had been pulled through the conduit. Additional inspection in the reactor enclosure identified a missing fire seal on elevation 352 feet near the reactor enclosure elevator around a power supply conduit to the elevator.

Consequences of the Event:

Fire barriers provide assurance that, in the event of a fire in any one fire area, a fire will not propagate to other areas and the plant can be safely shutdown. The above deviations were identified with the unit in the shutdown condition prior to initial criticality, and therefore, safety consequences are minimal.

Cause of the Event:

In the case of the seismic gap seal, the cause of the event appears to be failure to re-install the seal after construction activities in the area of the fire damper. In the case of use of the spare conduit, the cause of the event appears to be the failure of plant personnel to recognize that use of the spare conduit would result in violation of a fire barrier.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1) Limerick Generating Station Unit 1	DOCKET NUMBER (2) 0 5 0 0 0 3 5 2	LER NUMBER (8)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		8 4	- 0 2 2	- 0 1 0	0 3	OF	0 3

TEXT (If more space is required, use additional NRC Form 308A) (17)

Corrective Actions:

When the fire penetrations were discovered not sealed a fire watch/patrol was posted in the affected areas within one hour in accordance with Technical Specification requirements. The fire detectors in the areas so equipped were verified operable.

All power ascension program test cables in the Reactor Enclosure were walked down and no additional fire barrier violations identified.

Additional inspection as a result of the initial discovery revealed an unsealed conduit penetration on elevation 352 feet in the reactor enclosure. This conduit penetration contained a power supply cable for the Unit 2 side reactor enclosure elevator.

All seismic gap fire barriers in the Reactor Enclosure were identified and approximately 2/3 have been inspected with no discrepancies identified. Inspection is ongoing and presently anticipated to be completed by January 7, 1984.

The conduit penetration seals have been replaced. The seismic gap fire seal at the fire damper on elevation 201 feet is being redesigned due to accessibility problems and is expected to be installed by February 1, 1985. A fire watch/patrol will remain in this area until this penetration seal is completed.

By way of letter from the Station Superintendent all affected organizations have been instructed in the Technical Specification requirements regarding the operability of fire barriers and penetration seals.

PHILADELPHIA ELECTRIC COMPANY

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December 28, 1984

Docket No. 50-352

Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, DC 20555

SUBJECT: Licensee Event Report
Limerick Generating Station - Unit 1

This Licensee Event Report concerns inoperable fire penetration seals in the reactor enclosure prior to initial criticality.

Reference: Docket No. 50-352
Report Number: 84-022
Revision Number: 00
Event Date: November 29, 1984
Report Date: December 28, 1984
Facility: Limerick Generating Station
P.O. Box A, Sanatoga, PA 19464

This LER is submitted pursuant to the requirements of 10CFR50.73 (a)(2)(i).

Very truly yours,

William C. Whitfield
W. T. Ullrich *for W.T.U.*
Superintendent
Nuclear Generation Division

cc: Dr. Thomas E. Murley, Administrator, Region I, USNRC
J. T. Wiggins, Senior Site Inspector
See Service List

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