NRC Form 366 (9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO 3150-0104
EXPIRES 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
ST. LUCIE UNIT 2	0 5 0 0 0 3 8 9	1 OF 0 3

TITLE (4) ISI SNUBBER INSPECTION FAILURES

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
1 1	1 9	8 4	8 4	— 0 1 0	— 0 0	1 2	1 9	8 4	N/A	0 5 0 0 0
										0 5 0 0 0

OPERATING MODE (9)	6	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)			
POWER LEVEL (10)	0 0 0	20.402(b)	20.405(c)	50.73(a)(2)(iv)	73.71(b)
		20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
		20.405(a)(1)(ii)	50.36(c)(2)	X 50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
		20.405(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
		20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
		20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
G. E. Walling - Technical Staff	AREA CODE 3 0 5 — 4 6 5 - 3 5 5 0

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	
X	A B	S N B	P 0 2 9	Y		X	S J	S N B	P 0 2 9	Y	
X	B P	S N B	P 0 2 9	Y							

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

During the first Unit 2 refueling outage the ISI surveillance requirements of Technical Specification 4.7.9 were performed. As a result of this inspection, 19 snubbers were declared inoperable with respect to the acceptance criteria. Of these 19, 12 were considered to be non-functional and 7 would have performed their intended function but marginally exceeded the specified G force value (.02g) for activation. All 19 inoperable snubbers have been replaced or repaired.

During the next scheduled surveillance, FPL intends to perform functional testing of snubbers in all locations where failures were encountered independent of the required 10% test sample.

There was no adverse effect on the health and safety of the public.

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ATTACHMENT A
TABULATION OF INOPERABLE SNUBBERS
1984 ISI

I ½, ¼ KIP design type declared inoperable and considered non-funtional based on manual stroking.

Snubber ID#	Type	Service	Failure	Corrective action
2-025	PSA-½	RCP Seal Pres	Locked Up	Replaced w/ same
2-026	PSA-½	RCP Seal Pres	Locked Up	Replaced w/ same
2-030	PSA-½	RCP P	Locked Up	Replaced w/ same
2-079	PSA-¼	SDC Crossover	Broken Lead Screw	Replaced w/ PSA-1
2-157	PSA-½	Porv outlet	Bent lead screw	Replaced w/ PSA-1
2-158	PSA-½	Porv outlet	Bent lead screw	Replaced w/ PSA-1

II ½, ¼ KIP design type declared inoperable by funtional test 10% test sample - no failures

III 1, 3, 10 design type declared inoperable by funtional test but considered to be funtional. (100% of the snubbers in this design category were functionally tested)

All of the snubbers listed below were declared inoperable based on acceleration activation values and were repaired or replaced with same.

Snubber ID#	Type	Service	As Found	Deviation
2-065	PSA-1	Reactor Drain Tk coutlet	0.026g	+.006g
2-091	PSA-10	"A" Main Feed	0.022g	+.002g
2-146	PSA-10	Per safety to Q.T.	0.026g	+.006g
2-162	PSA-10	Per Relief	0.026g	+.006g
2-173	PSA-10	Per safety to Q.T.	0.022g	+.002g
2-217	PSA-10	RWT to "B"train	0.023g	+.003g
2-312	PSA-10	"B" Main Feed	0.024g	+.004g

Continued:

IV 1, 3, 10 KIP design type declared inoperable by funtional test and considered to be non-funtional

100% Test Sample

Snubber ID#	Type	Service	Failure	Corrective Action
2-068	PSA-10	Sit to loop	Binding	Replaced w/ Hydraulic
2-070	PSA-10	Sit to loop	Locked up	Replaced w/ Hydraulic
2-105	PSA-10	Sit to loop	Locked up	Replaced with same
2-167	PSA-3	PZR Relief	Failed running Drag	Replaced w/ Hydraulic
2-175	PSA-1	PZR Relief to QT	Capstan spring broken	Replaced with same
2-200	PSA-1	Ck vlv leak to RWT	Capstan spring broken	Replaced with same

FLORIDA POWER & LIGHT COMPANY

December 26, 1984
L-84-381

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555

Gentlemen:

Re: Reportable Event 84-10
St. Lucie Unit 2
Date of Event: November 19, 1984
Snubber Inspection

The attached Licensee Event Report is being submitted pursuant to the requirements of 10 CFR to provide notification of the subject event.

Very truly yours,

J. W. Williams, Jr.
Group Vice President
Nuclear Energy

JWW/PKG/js

Attachment

cc: J. P. O'Reilly, Region II, USNRC
Harold F. Reis, Esquire
PNS-LI-84-459-1

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