

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) DONALD C. COOK - UNIT 1										DOCKET NUMBER (2) 0 5 0 0 0 0 0 0 0 0				PAGE (3) 1 OF 02					
TITLE (4) IMPROPERLY CLASSIFIED FIRE DAMPERS																			
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)									
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)						
0	2	0	6	8	5	8	5	0	0	6	0	0	0	3	1	6			
0 2 0 6 8 5 8 5 0 0 6 0 0 0 3 0 8 8 5																			
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)																	
1		20.402(b)				20.405(c)				50.73(a)(2)(iv)				73.71(b)					
POWER LEVEL (10)		20.406(a)(1)(i)				50.36(c)(1)				50.73(a)(2)(v)				73.71(c)					
0 7 0		20.406(a)(1)(ii)				50.36(c)(2)				50.73(a)(2)(vii)				OTHER (Specify in Abstract below and in Text, NRC Form 386A)					
		20.406(a)(1)(iii)				X 50.73(a)(2)(ii)				50.73(a)(2)(viii)(A)									
		20.406(a)(1)(iv)				50.73(a)(2)(iii)				50.73(a)(2)(viii)(B)									
		20.406(a)(1)(v)				50.73(a)(2)(iii)				50.73(a)(2)(x)									
LICENSEE CONTACT FOR THIS LER (12)																			
NAME N. C. WILLIAMS - MAINTENANCE DEPARTMENT SUPERINTENDENT										TELEPHONE NUMBER 6 1 6 4 6 5 5 9 0 1									
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																			
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC									
X	K/Q	B/D/M	P/A	3/4	0	N													
SUPPLEMENTAL REPORT EXPECTED (14)												EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR			
X YES (If yes, complete EXPECTED SUBMISSION DATE)												NO		0	5	0	1	8	5

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

ON 02/06/85 AT 1300 HOURS WITH UNIT 1 REACTOR IN MODE 1 (POWER OPERATION) AT 70 PERCENT POWER AND UNIT 2 REACTOR IN MODE 1 AT 100 PERCENT POWER, AN AUXILIARY BUILDING ACCESS CONTROL AREA VENTILATION DUCT FIRE DAMPER WAS FOUND CLOSED DURING THE PERFORMANCE OF MAINTENANCE ACTIVITIES IN THE AREA. INITIAL INVESTIGATION OF THE CONDITION DETERMINED THAT THE FIRE DAMPER WAS DESIGNED TO PERFORM A SAFETY FUNCTION BUT HAD BEEN MISIDENTIFIED AS NON-SAFETY RELATED DURING THE SAFETY RELATED FIRE DAMPER IDENTIFICATION PROGRAM OF 1982. AS A RESULT OF THE IMPROPER IDENTIFICATION, THE DAMPER WAS RESET AND SURVEILLANCE TESTED PURSUANT TO TECHNICAL SPECIFICATION 4.7.10.3 AT 0817 HOURS ON 02/07/85.

A CONTINUING INVESTIGATION INTO THIS EVENT INDICATED THAT FOUR ADDITIONAL FIRE DAMPERS WERE NOT INCLUDED IN THE TABULATION OF SAFETY RELATED FIRE DAMPERS. ALL FIVE (5) FIRE DAMPERS ARE BEING ADDED TO THE FIRE DAMPER SURVEILLANCE TEST SCHEDULE. A COMPREHENSIVE REVIEW OF PLANT FIRE DAMPERS IS CURRENTLY BEING PERFORMED. A FOLLOW-UP REPORT WILL BE SUBMITTED WHEN THIS REVIEW IS COMPLETE.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/85

FACILITY NAME (1) DONALD C. COOK - UNIT 1	DOCKET NUMBER (2) 0 5 0 0 0 3 1 5 8 5	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		8 5	0 0 6	0 0 0	2	OF	0 2

TEXT (If more space is required, use additional NRC Form 386A's) (17)

ON 02/06/85 AT 1300 HOURS WITH UNIT 1 (POWER OPERATION) AT 70 PERCENT POWER AND UNIT 2 REACTOR IN MODE 1 AT 100 PERCENT POWER, A CLOSED FIRE DAMPER (IEEE/BDMP) WAS DISCOVERED DURING MAINTENANCE ACTIVITIES BEING PERFORMED ON THE AUXILIARY BUILDING ACCESS CONTROL AREA VENTILATION SYSTEM. INITIAL INVESTIGATION OF THE CONDITION DETERMINED THAT THE FIRE DAMPER WAS DESIGNED TO PERFORM A SAFETY FUNCTION BUT HAD BEEN MISIDENTIFIED AS NON-SAFETY RELATED DURING THE SAFETY RELATED FIRE DAMPER IDENTIFICATION PROGRAM OF 1982. AS A RESULT OF THE IMPROPER IDENTIFICATION, THE DAMPER WAS NOT IN THE FIRE DAMPER SURVEILLANCE TEST SCHEDULE. A FIRE WATCH WAS POSTED AT THE TIME THE DAMPER WAS VERIFIED TO BE SAFETY RELATED.

A MAKEUP AIR PREHEATER STEAM COIL IS LOCATED IN THE VENTILATION DUCT, JUST UPSTREAM OF THE FIRE DAMPER. IT HAS BEEN CONCLUDED THAT A VENTILATION SYSTEM MALFUNCTION ALLOWED A HEAT BUILDUP IN THE DUCT WHICH CAUSED SEPARATION OF THE AUTOMATIC ACTUATION THERMAL LINKS WHICH HELD THE DAMPER IN THE OPEN POSITION. SINCE THE DAMPER ACTUATED AS DESIGNED, NO ADVERSE SAFETY CONSEQUENCES WOULD HAVE RESULTED FROM THE FAILURE TO TEST THIS DAMPER. THE DAMPER WAS SERVICED AND RESET. POST-MAINTENANCE SURVEILLANCE TESTING IN COMPLIANCE WITH TECHNICAL SPECIFICATION 4.7.10.3 WAS COMPLETED SATISFACTORILY PER **12MHP4030.STP.028 at 0817 HOURS ON 02/07/85.

A CONTINUING INVESTIGATION INTO THIS EVENT HAS INDICATED THAT FOUR ADDITIONAL FIRE DAMPERS THAT PERFORM SAFETY FUNCTIONS WERE NOT PROPERLY IDENTIFIED AS SAFETY RELATED AND, THEREFORE, WERE NOT SURVEILLANCE TESTED PRIOR TO THIS DISCOVERY. ALL FIVE FIRE DAMPERS ARE BEING ADDED TO THE SAFETY RELATED FIRE DAMPER SURVEILLANCE TEST PROGRAM.

A COMPREHENSIVE REVIEW OF PLANT FIRE DAMPERS IS CURRENTLY BEING PERFORMED. THIS REVIEW WILL INCLUDE AN ATTEMPT TO OBTAIN ADDITIONAL INFORMATION PERTAINING TO THE OMISSION OF THE SUBJECT DAMPERS FROM THE SURVEILLANCE TEST PROGRAM. A FOLLOW-UP REPORT WILL BE SUBMITTED WHEN THIS REVIEW IS COMPLETE. THE FOLLOW-UP REPORT WILL ALSO CONTAIN AN ENGINEERING EVALUATION OF THE SAFETY SIGNIFICANCE OF THE FOUR DAMPERS NOT HAVING BEEN TESTED PREVIOUS TO THEIR RECENT DESIGNATION AS SAFETY RELATED.



INDIANA & MICHIGAN ELECTRIC COMPANY

DONALD C. COOK NUCLEAR PLANT
P.O. Box 458, Bridgman, Michigan 49106
(616) 465-5901

March 8, 1985

United States Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

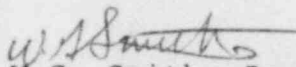
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Docket No. 50-315

Document Control Manager:

In accordance with the criteria established by 10CFR50.73
entitled Licensee Event Reporting System, the following
report/s are being submitted:

RO 85-006-0

Sincerely,


W.G. Smith, Jr.
Plant Manager

/cbm

Attachment

cc: John E. Dolan
J.G. Keppler, RO:III
M.P. Alexich
R.F. Kroeger
H.B. Brugger
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