

LASALLE NUCLEAR POWER STATION

UNIT 1

MONTHLY PERFORMANCE REPORT

FEBRUARY 1985

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-373

LICENSE NO. NPF-11

8503120357 850228
PDR ADDOCK 05000373
R PDR

TABLE OF CONTENTS

- I. INTRODUCTION
- II. MONTHLY REPORT FOR UNIT ONE
 - A. Summary of Operating Experience
 - B. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS, AND SAFETY RELATED MAINTENANCE
 - 1. Amendments to Facility License or Technical Specifications
 - 2. Facility or Procedure Changes Requiring NRC Approval
 - 3. Tests and Experiments Requiring NRC Approval
 - 4. Corrective Maintenance of Safety Related Equipment
 - C. LICENSEE EVENT REPORTS
 - D. DATA TABULATIONS
 - 1. Operating Data Report
 - 2. Average Daily Unit Power Level
 - 3. Unit Shutdowns and Power Reductions
 - E. UNIQUE REPORTING REQUIREMENTS
 - 1. Main Steam Relief Valve Operations
 - 2. ECCS System Outages
 - 3. Off-Site Dose Calculation Manual Changes
 - 4. Major Changes to Radioactive Waste Treatment System

I. INTRODUCTION

The LaSalle Nuclear Power Station is a Two Unit Facility Located in Marseilles, Illinois. Each Unit is a Boiling Water Reactor with a designed electrical output of 1078 MWe net. The Station is owned by Commonwealth Edison Company. The Architect/Engineer was Sargent & Lundy, and the primary construction contractor was Commonwealth Edison Company.

The condenser cooling method is a closed cycle cooling pond. Unit One is subject to License Number NPF-11, issued on April 17, 1982. The date of initial criticality was June 21, 1982. Unit Two is subject to license number NPF-18, issued on December 16, 1983. The date of initial criticality was March 10, 1984.

This report was compiled by Richard J. Rohrer telephone number (815)357-6761, extension 575.

II. MONTHLY REPORT FOR UNIT ONE

A. SUMMARY OF OPERATING EXPERIENCE FOR UNIT ONE

FEBRUARY 1-2

Feb. 1, 0001 hours - Unit on-line at 90% reactor power.

Feb. 2, 1140 hours - Power reduction commenced because 1VR05YA and B were found inoperable.

Feb. 2, 1200 hours - Main Steam Isolation valves closed causing a unit scram and SRV actuations. The isolation signal was High Steam Tunnel Ambient Temperature. The reactor was critical for 36 hours and 0 minutes.

FEBRUARY 3-9

Feb. 3, 1714 hours - Reactor went critical.

Feb. 4, 0600 hours - Generator synchronized to the grid.

Feb. 8, 1700 hours - Reactor power at 78%.

Feb. 8, 1018 hours - Feedwater pumps tripped on low suction pressure causing reactor scram on low level.

The reactor had been critical for 149 hours and 4 minutes so far for February.

FEBRUARY 10-28

Feb. 10, 0115 hours - Reactor went critical.

Feb. 10, 1013 hours - Generator synchronized to the grid.

Feb. 12, 0700 hours - Reactor power at 83%.

Feb. 13, 0030 hours - Load reduction begun in order to manipulate control rods.

Feb. 13, 0320 hours - Reactor power at 55%.

Feb. 13, 0700 hours - Reactor power at 71%.

Feb. 22, 2300 hours - Power reduction caused by moisture separator/reheater problems.

Feb. 23, 0700 hours - Reactor power at 92%.

Feb. 23, 2200 hours - Load reduction begun in order to manipulate control rods.

Feb. 24, 0450 hours - Reactor power at 59%.

Feb. 26, 0700 hours - Reactor power at 98%.

Feb. 28, 2300 hours - Reactor power at 99%. The reactor was critical for a total of 603 hours and 49 minutes in February, 1985.

B. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY RELATED MAINTENANCE.

1. Amendments to facility license or Technical Specification.

There were no amendments to the facility license or Technical Specification during this reporting period.

2. Facility or procedure changes requiring NRC approval.

There were no facility or procedure changes requiring NRC approval during this reporting period.

3. Tests and Experiments requiring NRC approval.

There were no tests or experiments requiring NRC approval during this reporting period.

4. Corrective maintenance of safety related equipment.

The following table (Table 1) presents a summary of safety-related maintenance completed on Unit One during the reporting period. The headings indicated in this summary include: Work Request number, Component Name, Cause of Malfunction, Results and Effects on Safe Operation, and Corrective Action.

TABLE 1

CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

WORK REQUEST	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS	CORRECTIVE ACTION ON SAFE OPERATION
L41788	Outside Air Damper for Switchgear HVAC System "A".	Damper jammed closed.	Potential for inadequate ventilation of switchgear.	Damper actuator was rebuilt.
L45595	Control Room Emergency Make-up flow Recorder.	Pen would not move properly due to slipping clutch.	Incorrect indication.	Clutch was cleaned.
L45989	Reactor Building HVAC exhaust damper 1VR05YA	Damper would not automatically close because of solenoid problem.	Taken together, the inability of dampers 1VR05YA and B to automatically close caused the inability to automatically close a penetration of the secondary containment. Local manual isolation was still possible.	Rebuilt solenoid. Replaced Solenoid
L45990	Reactor building HVAC Exhaust damper 1VR05YB	Damper would not automatically close because of solenoid problem		
L45993	Safety/Relief Valve "K"	Setpoint driftactuated at too low a pressure.	Third SR/V opened at a slightly lower pressure than design.	Recalibrated.
L45996	Outboard Main Steam Isolation Valve "A"	Limit Switch out of position no full-open indication.	Loss of position indication	Adjusted limit switch.
L45997	Automatic Depressurization system valve "S" accumulator low pressure alarm.	Out-of-calibration switches	Continuous Alarm.	Recalibrated switches.
L46146	Chlorine Detector "A"	Did not function properly, bad diode.	Loss of redundancy for ESF actuation.	Replaced bad diode.

TABLE I

CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

WORK REQUEST	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS	CORRECTIVE ACTION ON SAFE OPERATION
L46193	RCIC Trip and throttle valve.	Would not open due to improperly engaged linkage arm.	Temporary inoperability of RCIC.	Linkage arm was properly engaged.
L46197	Safety/Relief Valve "G"	Opened at too low a pressure due to setpoint drift.	Relieved Reactor Pressure at a pressure slightly below design.	Recalibrated.
L46231	Narrow Range Reactor Pressure Recorder.	Recorder was out of calibration.	Incorrect indication.	Recalibrated.
L46361	Rod Block Monitor "B"	Gain card 231 was defective	Loss of ability to adjust RBM gain.	Repaired and recalibrated gain card.
L46479	RCIC Water Leg Pump	Worn Oil Seal	Excessive Oil Leakage requiring frequent filling.	Replaced seal.

C. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for LaSalle Nuclear Power Station, Unit One, occurring during the reporting period, February 1 through February 28, 1985. This information is provided pursuant to the reportable occurrence reporting requirements as set forth in 10CFR 50.73.

<u>Licensee Event Report Number</u>	<u>Date</u>	<u>Title of Occurrence</u>
85-004-00	1-12-85	Unsecured High Rad Door
85-005-00	1-14-85	Both Standby Gas Treatment Trains Inoperable
85-006-00	1-12-85	LIS-NB-104 Time Clock Exceeded.
85-007-00	1-10-85	Control Room HVAC Chloride Detector Spurious Trip.
85-008-00	2-2-85	1VR05YA and B Dampers Would Not Close.
85-009-00	1-22-85	Missed Technical Specification Surveillance Requirement.
85-010-00	1-21-85	B Control Room HVAC Ammonia Detector Actuation.
85-011-00	2-2-85	Scram Due to MSIV Isolation.
85-012-00	2-8-85	Reactor Water Cleanup Isolation
85-013-00	2-12-85	Reactor Water Cleanup Isolation
85-014-00	2-4-85	Drywell Vacuum Breaker Opening (ESF Actuation)

D. DATA TABULATIONS

The following data tabulations are presented in this report:

1. Operating Data Report
2. Average Daily Unit Power Level
3. Unit Shutdowns and Power Reductions

1. OPERATING DATA REPORT

DOCKET NO. 050-373
 UNIT LaSalle One
 DATE March 10, 1985
 COMPLETED BY Richard J. Rohrer
 TELEPHONE (815)357-6761

OPERATING STATUS

1. REPORTING PERIOD: February, 1985 GROSS HOURS IN REPORTING PERIOD: 672
 2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 3323 MAX DEPEND CAPACITY
 (MWe-Net): 1036 DESIGN ELECTRICAL RATING (MWe-Net): 1078
 3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A
 4. REASONS FOR RESTRICTION (IF ANY): N/A

	THIS MONTH	YR TO DATE	CUMULATIVE
5. NUMBER OF HOURS REACTOR WAS CRITICAL	<u>603.8</u>	<u>1318.3</u>	<u>7599</u>
6. REACTOR RESERVE SHUTDOWN HOURS	<u>0.0</u>	<u>29.5</u>	<u>1195</u>
7. HOURS GENERATOR ON LINE	<u>582.1</u>	<u>1277.6</u>	<u>7333</u>
8. UNIT RESERVE SHUTDOWN HOURS	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
9. GROSS THERMAL ENERGY GENERATED (MWH)	<u>1618498</u>	<u>3502611</u>	<u>20325900</u>
10. GROSS ELEC. ENERGY GENERATED (MWH)	<u>536590</u>	<u>1154878</u>	<u>6625521</u>
11. NET ELEC. ENERGY GENERATED (MWH)	<u>515541</u>	<u>1109936</u>	<u>6304998</u>
12. REACTOR SERVICE FACTOR	<u>89.9%</u>	<u>93.1%</u>	<u>74.5%</u>
13. REACTOR AVAILABILITY FACTOR	<u>89.9%</u>	<u>95.2%</u>	<u>86.2%</u>
14. UNIT SERVICE FACTOR	<u>86.6%</u>	<u>90.2%</u>	<u>71.9%</u>
15. UNIT AVAILABILITY FACTOR	<u>86.6%</u>	<u>90.2%</u>	<u>71.9%</u>
16. UNIT CAPACITY FACTOR (USING MDC)	<u>74.1%</u>	<u>75.7%</u>	<u>59.7%</u>
17. UNIT CAPACITY FACTOR (USING DESIGN MWe)	<u>71.2%</u>	<u>72.7%</u>	<u>57.3%</u>
18. UNIT FORCED OUTAGE RATE	<u>13.4%</u>	<u>9.8%</u>	<u>14.2%</u>
19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH)			
20. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: <u>N/A</u>			

2. AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO: 050-373
UNIT: LASALLE ONE
DATE: March 10, 1985
COMPLETED BY: Richard J. Rohrer
TELEPHONE: (815) 357-6761

MONTH: FEBRUARY, 1985

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

1. _____ 998 _____	17. _____ 1056 _____
2. _____ 504 _____	18. _____ 1007 _____
3. _____ 0 _____	19. _____ 1072 _____
4. _____ 171 _____	20. _____ 1065 _____
5. _____ 511 _____	21. _____ 1062 _____
6. _____ 783 _____	22. _____ 1043 _____
7. _____ 713 _____	23. _____ 949 _____
8. _____ 345 _____	24. _____ 684 _____
9. _____ 0 _____	25. _____ 872 _____
10. _____ 149 _____	26. _____ 1051 _____
11. _____ 716 _____	27. _____ 1028 _____
12. _____ 904 _____	28. _____ 1079 _____
13. _____ 757 _____	29. _____ NA _____
14. _____ 962 _____	30. _____ NA _____
15. _____ 980 _____	31. _____ NA _____
16. _____ 1055 _____	

ATTACHMENT E

3. UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 050-373UNIT NAME LaSalle OneDATE March 10, 1985COMPLETED BY Richard J. RohrerTELEPHONE (815)357-6761REPORT MONTH February 1985

NO.	DATE	TYPE		DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS
		F: FORCED	S: SCHEDULED				
3	850202	F		42.00	A	3	Unit scrammed following MSIV Isolation on High Steam Tunnel Ambient Temperature. Reported in LER 85-022-00.
4	850208	F		47.92	A	3	Unit scrammed on low vessel level following trip of Feedwater Pumps on Low Suction Pressure.

E. UNIQUE REPORTING REQUIREMENTS

1. Safety/Relief valve operations for Unit One.

<u>DATE</u>	<u>VALVES ACTUATED</u>	<u>NO & TYPE ACTUATION</u>	<u>PLANT CONDITION</u>	<u>DESCRIPTION OF EVENT</u>
2-2-85	1B21-F012U	16 Automatic	1075 psig	Opened repeatedly to relieve reactor pressure following Main Steam Line Isolation.
2-2-85	1B21-F013S	1 Automatic	1075 psig	Opened to relieve reactor pressure following Main Steam Line Isolation.
2-2-8J	1B21-F013K	1 Automatic	1075 psig	Opened to relieve reactor pressure following Main Steam Line Isolation.
2-8-85	1B21-F013E	2 Automatic	1075 psig	Cycled to relieve pressure following Main Steam Line Isolation.

2. ECCS Systems Outages

The following outages were taken on ECCS Systems during the reporting period.

<u>OUTAGE NO.</u>	<u>EQUIPMENT</u>	<u>PURPOSE OF OUTAGE</u>
1-93-85	1E12-F068A	Repack Valve.
1-95-85	1B Diesel Generator	Replace Relay Tachometewr.
1-96-85	1B Diesel Generator	Clean Lubricating Oil Separator Ejector Screen.
1-112-85	1E12-F068A	Repack Valve.

3. Off-Site Dose Calculation Manual

There were no changes to the off-site dose calculations manual during this reporting period.

4. Radioactive Waste Treatment Systems.

There were no changes made to the radioactive waste treatment system during this reporting period.

LASALLE NUCLEAR POWER STATION

UNIT 2

MONTHLY PERFORMANCE REPORT

FEBRUARY 1985

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-374

LICENSE NO. NPF-18

TABLE OF CONTENTS

- I. INTRODUCTION
- II. MONTHLY REPORT FOR UNIT TWO
 - A. Summary of Operating Experience
 - B. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS, AND SAFETY RELATED MAINTENANCE
 - 1. Amendments to Facility License or Technical Specifications
 - 2. Facility or Procedure Changes Requiring NRC Approval
 - 3. Tests and Experiments Requiring NRC Approval
 - 4. Corrective Maintenance of Safety Related Equipment
 - C. LICENSEE EVENT REPORTS
 - D. DATA TABULATIONS
 - 1. Operating Data Report
 - 2. Average Daily Unit Power Level
 - 3. Unit Shutdowns and Power Reductions
 - E. UNIQUE REPORTING REQUIREMENTS
 - 1. Safety/Relief Valve Operations
 - 2. ECCS System Outages
 - 3. Off-Site Dose Calculation Manual Changes
 - 4. Major Changes to Radioactive Waste Treatment System

I. INTRODUCTION

The LaSalle Nuclear Power Station is a Two Unit Facility Located in Marseilles, Illinois. Each Unit is a Boiling Water Reactor with a designed electrical output of 1078 MWe net. The Station is owned by Commonwealth Edison Company. The Architect/Engineer was Sargent & Lundy, and the primary construction contractor was Commonwealth Edison Company.

The condenser cooling method is a closed cycle cooling pond. Unit One is subject to License Number NPF-11, issued on April 17, 1982. The unit commenced commercial generation of power on January 1, 1984. Unit Two is subject to license number NPF-18, issued on December 16, 1983. The date of initial criticality was March 10, 1984.

This report was compiled by Richard J. Rohrer, telephone number (815)357-6761, extension 575.

II. MONTHLY REPORT FOR UNIT TWO

A. SUMMARY OF OPERATING EXPERIENCE FOR UNIT TWO

JANUARY 1-31

- Feb. 1, 0001 hours - Unit on-line at 98% reactor power.
- Feb. 4, 1530 hours - Power reduction due to Turbine Driven Reactor Feed Pump Problems; 83% reactor power.
- Feb. 5, 0700 hours - Reactor power restored to 93%. Following work on Turbine Driven Reactor Feed Pumps.
- Feb. 7, 2215 hours - Reactor Power reduced to 64% due to 2B Reactor Recirculation Flow Control Valve Ramping to its minimum position.
- Feb. 10, 2300 hours - Reactor Power at 95%.
- Feb. 17, 0900 hours - Power reduction commenced because of inoperable control rods.
- Feb. 17, 1850 hours - Power reduction ceased when all control rods operable. Reactor power at 73%.
- Feb. 19, 0700 hours - Reactor Power at 100%.
- Feb. 27, 1100 hours - Power reduction begun for scheduled outage.
- Feb. 28, 0520 hours - Generator manually tripped for scheduled outage.
- Feb. 28, 0749 hours - The reactor was manually scrammed for scheduled outage. The reactor was critical for 655 hours and 49 minutes.

B. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY RELATED MAINTENANCE.

1. Amendments to facility license or Technical Specifications.

There were no amendments to the facility license or Technical Specifications during this reporting period.

2. Facility or procedure changes requiring NRC approval.

There were no facility or procedure changes requiring NRC approval during the reporting period.

3. Tests and experiments requiring NRC approval.

There were no tests or experiments requiring NRC approval during the reporting period.

4. Corrective Maintenance of Safety Related Equipment.

The following table (Table 1) presents a summary of safety-related maintenance completed on Unit One during the reporting period. The headings indicated in this summary include: Work Request number, Component Name, cause of malfunction, results and effects on safe operation, and corrective action.

TABLE 1

CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

WORK REQUEST	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS	CORRECTIVE ACTION ON SAFE OPERATION
L44411	HPCS Water Leg Pump Discharge Stop Check Valve.	Valve Apparently did not fully open.	Potential to limit flow from jockey pump.	Valve cleaned and tested.
L44#12	HPCS Waterleg Pump Discharge Check Valve.	Valve Stuck Partially Closed.	Could not check flow properly.	Valve cleaned and tested.
L45729	Division 2 Wide Range Reactor Level Recorder	Alarm Setpoint out of adjustment.	Continuous Alarm.	Adjusted alarm and calibrated recorder.
L45813	Suppression Chamber Pressure Indicator.	Out-of-calibration	Incorrect indication	Recalibrated.
L45890	Average Power Range Monitor "F".	Blown fuse	No downscale light to indicate that LPRM F had been bypassed.	Replaced fuse.
L46103	2ARHR Service Water Pump	Outboard bearing leaked oil.	Need to replenish oil often.	Replaced bearing.
L46508	HCU Accumulator for CRD 34-51.	Leaking isolation Valve.	Potential to cause failure to scram this rod if combined with other events.	Replaced valve.
L46509	HCU Accumulator for CRD 30-55.	Leaking isolation valve.	Potential to cause failure to scram this rod if combined with other events.	Replaced valve.
L46511	HCU Accumulator for CRD 22-51.	Leaking Nitrogen drain valve.	Potential to cause failure to scram this rod if combined with other events.	Replaced valve.

TABLE 1

CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

WORK REQUEST	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS	CORRECTIVE ACTION ON SAFE OPERATION
L46565	HCU Accumulator for CRD 50-39.	Leaking instrument stop valve.	Potential to cause failure to scram this rod if combined with other events.	Replaced valve.
L46665	HCU Accumulator for CRD 38-11.	Leaking instrument isolation valve.	Potential to cause failure to scram this rod if combined with other events.	Replaced valve.
L46828	HCU Accumulator for CRD 18-43.	Leaking instrument block valve.	Potential to cause failure to scram this rod if combined with other events.	Replaced valve.
L46835	HCU Accumulator for CRD 10-39.	Leaking Isolation valve.	Potential to cause failure to scram this rod if combined with other events.	Replaced valve.
L46943	RCIC Area High Differential Temp- erature instrument "A"	Broken lead.	Loss of isolation function of this instrument. Redundant instruments were operable.	Repaired lead.

C. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for LaSalle Nuclear Power Station, Unit Two, occurring during the reporting period, February 1 through February 28, 1985. This information is provided pursuant to the reportable occurrence reporting requirements as set forth in 10CFR 50.73.

<u>Licensee Event Report Number</u>	<u>Date</u>	<u>Title of Occurrence</u>
85-002-00	1-13-85	HPCS Suction Swapped from CY to SF System.
85-003-00	1-12-85	Mised ODCM Calculation.
85-004-00	1-14-85	GSEP Alert - Unit Two, Plant Shutdown Required by Technical Specifications.
85-005-00	1-21-85	2WR040 Tripped Closed.
85-006-00	1-22-85	HPCS Suction Valves Swapped on Suppression Pool High Level.
85-007-00	2-5-85	Reactor Vessel High Water Level HPCS Injection Valve Closure.

D. DATA TABULATIONS

The following data tabulations are presented in this report:

1. Operating Data Report
2. Average Daily Unit Power Level
3. Unit Shutdowns and Power Reductions

1. OPERATING DATA REPORT

DOCKET NO. 050-374
 UNIT LaSalle Two
 DATE March 10, 1985
 COMPLETED BY Richard J. Rohrer
 TELEPHONE (815)357-6761

OPERATING STATUS

1. REPORTING PERIOD: February 1985 GROSS HOURS IN REPORTING PERIOD: 672
 2. CURRENTLY AUTHORIZED POWER LEVEL (Mwt): 3323 MAX DEPEND CAPACITY (MWe-Net): 1036 DESIGN ELECTRICAL RATING (MWe-Net): 1078
 3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A
 4. REASONS FOR RESTRICTION (IF ANY):
- | | THIS MONTH | YR TO DATE | CUMULATIVE |
|---|----------------|----------------|----------------|
| 5. NUMBER OF HOURS REACTOR WAS CRITICAL | <u>655.8</u> | <u>1399.8</u> | <u>3011.6</u> |
| 6. REACTOR RESERVE SHUTDOWN HOURS | <u>0.0</u> | <u>0.0</u> | <u>125.3</u> |
| 7. HOURS GENERATOR ON LINE | <u>653.3</u> | <u>1397.3</u> | <u>2934.7</u> |
| 8. UNIT RESERVE SHUTDOWN HOURS | <u>0.0</u> | <u>0.0</u> | <u>0.0</u> |
| 9. GROSS THERMAL ENERGY GENERATED (MWH) | <u>2020178</u> | <u>4382385</u> | <u>8894977</u> |
| 10. GROSS ELEC. ENERGY GENERATED (MWH) | <u>674930</u> | <u>1460378</u> | <u>2945373</u> |
| 11. NET ELEC. ENERGY GENERATED (MWH) | <u>651757</u> | <u>1410977</u> | <u>2803094</u> |
| 12. REACTOR SERVICE FACTOR | <u>97.6%</u> | <u>98.9%</u> | <u>94.4%</u> |
| 13. REACTOR AVAILABILITY FACTOR | <u>97.6%</u> | <u>98.9%</u> | <u>98.3%</u> |
| 14. UNIT SERVICE FACTOR | <u>97.2%</u> | <u>98.7%</u> | <u>92.0%</u> |
| 15. UNIT AVAILABILITY FACTOR | <u>97.2%</u> | <u>98.7%</u> | <u>92.0%</u> |
| 16. UNIT CAPACITY FACTOR (USING MDC) | <u>93.6%</u> | <u>96.2%</u> | <u>84.8%</u> |
| 17. UNIT CAPACITY FACTOR (USING DESIGN MWe) | <u>90.0%</u> | <u>92.4%</u> | <u>81.5%</u> |
| 18. UNIT FORCED OUTAGE RATE | <u>0.0%</u> | <u>0.0%</u> | <u>7.6%</u> |
19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):
 An outage for maintenance and surveillance was begun at 0520 on February 28, 1985.
 20. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP 5/13/85

2. AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO: 050-374
UNIT: LASALLE TWO
DATE: March 10, 1985
COMPLETED BY: Richard J. Rohrer
TELEPHONE: (815) 357-6761
MONTH: February 1985

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

1. _____	1055	17. _____	955
2. _____	1057	18. _____	936
3. _____	1040	19. _____	1071
4. _____	945	20. _____	1075
5. _____	1010	21. _____	1075
6. _____	1024	22. _____	1062
7. _____	993	23. _____	1069
8. _____	623	24. _____	1069
9. _____	776	25. _____	1071
10. _____	971	26. _____	1071
11. _____	1027	27. _____	811
12. _____	1063	28. _____	40
13. _____	1063	29. _____	NA
14. _____	1068	30. _____	NA
15. _____	1061	31. _____	NA
16. _____	1076		

ATTACHMENT E

3. UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 050-374UNIT NAME LaSalle TwoDATE MARCH 10, 1985COMPLETED BY Richard J. RohrerTELEPHONE (815)357-6761REPORT MONTH FEBRUARY 1985

NO.	DATE	TYPE		DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS
		F: FORCED	S: SCHEDULED				
2	850207	F		0.0	A	4	Reactor Recirculation Flow Control Valve "B" ramped to its minimum position causing the reduction in power.
3	850228	S		18.7	B	2	Manually shutdown for Maintenance and Surveillance outage.

E. UNIQUE REPORTING REQUIREMENTS

1. Safety/Relief Valve Operations for Unit Two.

<u>DATE</u>	<u>VALVES</u> <u>ACTUATED</u>	<u>NO & TYPE</u> <u>ACTUATIONS</u>	<u>PLANT</u> <u>CONDITION</u>	<u>DESCRIPTION</u> <u>OF EVENT</u>
-------------	----------------------------------	---	----------------------------------	---------------------------------------

There were no safety relief valve actuators during this reporting period.

2. ECCS Systems Outages

The following outages were taken on ECCS Systems during the reporting period.

<u>OUTAGE NO.</u>	<u>EQUIPMENT</u>	<u>PURPOSE OF OUTAGE</u>
2-122-85	2A RHR Service Water Pump	Repair Oiler
2-123-85	2A RHR Service Water Pump	Replace Bearing
2-129-85	2B Diesel Generator	Lubrication
2-130-85	2B Diesel Generator	Lubrication
2-133-85	2B Diesel Generator Air Compressor	Lubrication
2-139-85	2A Diesel Generator	Lubrication
2-140-85	2A Diesel Generator	Lubrication
2-145-85	2A Diesel Generator Air Compressor	Replace Gasket
2-148-85	High Pressure Core Core Spray	Repair Valves 2E22-F006 and 2E22-F007.
2-150-85	High Pressure Core Spray	Install and Remove Flushing Spool Piece
2-155-85	2B RHR Pump	Lubrication and sample.

3. Off-Site Dose Calculation Manual

There were no changes to the off-site dose calculations manual during this reporting period.

4. Radioactive Waste Treatment Systems.

There were no changes made to the radioactive waste treatment system during this reporting period.



Commonwealth Edison
LaSalle County Nuclear Station
Rural Route #1, Box 220
Marseilles, Illinois 61341
Telephone 815/357-6761

March 10, 1985

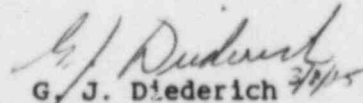
Director, Office of Management Information
and Program Control
United States Nuclear Regulatory Commission
Washington, D.C. 20555

ATTN: Document Control Desk

Gentlemen:

Enclosed for your information is the monthly performance report covering LaSalle County Nuclear Power Station for the period covering February 1 through February 28, 1985.

Very truly yours,


G. J. Diederich ^{3/11/85}
Superintendent
LaSalle County Station

GJD/RSD/crh

Enclosure

xc: J. G. Keppler, NRC, Region III
NRC Resident Inspector LaSalle
Gary Wright, Ill. Dept. of Nuclear Safety
D. P. Galle, CECO
D. L. Farrar, CECO
INPO Records Center
Ron A. Johnson, PIP Coordinator SNED
J. E. Ellis, GE Resident
J. M. Nowicki, Asst. Comptroller

IE2A
1/1