



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

May 2, 1996

Mr. Douglas R. Gipson
Senior Vice President
Nuclear Generation
Detroit Edison Company
6400 North Dixie Highway
Newport, Michigan 48166

SUBJECT: FERMI 2 - SAFETY EVALUATION OF PUMP AND VALVE RELIEF REQUESTS FOR
THE PUMP AND VALVE INSERVICE TESTING PROGRAM (TAC NO. M91392)

Dear Mr. Gipson:

During an NRC inservice testing (IST) inspection conducted at Fermi on January 23 - 27, 1995, the inspectors identified a number of relief requests in a letter dated September 20, 1985, which had not been reviewed by the staff. Your September 20, 1985, letter submitted revisions to the IST program for pumps and valves including requests for relief from the requirements of the American Society of Mechanical Engineers (ASME) Code. The relief requests identified in the September 20, 1985, letter include PR-6, PR-7, PR-8, VR-45, VR-46, VR-47, VR-48, VR-49, VR-50 and VR-51. Relief requests PR-6, PR-7 and PR-8 were subsequently revised in a letter dated July 14, 1995. Your July 14, 1995, letter also submitted an additional relief request, PR-11.

In addition, Revision 3, Change 9 to your IST program was submitted in a letter dated July 19, 1995. Revision 3, Change 9 to your IST program included the following changes to the previously submitted relief requests: (1) VR-46, VR-47 and VR-49 were revised; (2) VR-45 and VR-48 were revised to meet the requirements of NRC Generic Letter 89-04, and (3) VR-50 was deleted. Relief request PR-7-R1 was subsequently withdrawn in a letter dated February 2, 1996, after you determined that individual testing of the residual heat removal service water (RHRSW) pumps was not impractical. You also stated in the February 2, 1996, letter that fixed reference values for the four RHRSW pumps would be developed and implemented by March 25, 1996. Relief requests VR-46, VR-47 and VR-49 have been further revised on various occasions.

Evaluations of requests for relief PR-6-R1, PR-8-R1, PR-11, VR-46-R5, VR-47-R2, VR-49-R3 and VR-51 are included in the attached safety evaluation. Provisional relief for PR-6-R1 is granted to perform parallel pump testing of each core spray pump train and to use reference curves to assess the performance of each parallel pump combination pursuant to 10 CFR 50.55a(f)(6)(i) based on the impracticality of performing the testing in accordance with the Code requirements, and in consideration of the burden on the licensee if the Code requirements were imposed on the facility. The provisions associated with PR-6-R1 are identified in Section 3.1.4 of the staff's safety evaluation.

Provisional relief request PR-8-R1 to use the vibration requirements of ASME/ANSI Operations and Maintenance Standard, Part 6 (OM-6) for vibration

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testing of the RHR pumps is approved pursuant to 10 CFR 50.55a(f)(4)(iv) provided that all the related requirements of OM-6 listed in Section 3.2.3 of the staff's safety evaluation are met. The requested relief to exclude the requirements of OM-6 Section 4.6.4(b) from your vibration testing of the RHR pumps is denied. You should apply all the vibration requirements of OM-6 to vibration testing of the RHR pumps. In addition, you identify the four RHR pumps at Fermi as vertical line shaft pumps. RHR pumps at boiling water reactors are typically vertical centrifugal pumps. The two types of pumps are classified differently in OM-6 with more stringent acceptance criteria assigned to the vertical line shaft pumps. You should resolve this discrepancy within your IST program and current and future program submittals as appropriate.

Provisional relief request PR-11 is granted pursuant to 10 CFR 50.55a(f)(6)(i) to verify Code compliance of the hydraulic parameters of the emergency equipment cooling water (EECW) pumps by using pump reference curves based on the impracticality of performing testing in accordance with the Code requirements, and in consideration of the burden on the licensee if the Code requirements were imposed on the facility. The relief is granted with the provision that the elements listed in NUREG-1482, Section 5.2, are addressed and documented in your IST program.

Valve relief request VR-46-R5 requested relief from the analysis of leak rate requirements of ASME Section XI, Paragraph IWV-3426, for 77 containment isolation valves (CIVs). VR-47-R2 requested relief from the analysis of leak rate requirements of ASME Section XI, Paragraph IWV-3426, for 14 CIVs. Both of these relief requests for leak rate testing of specific CIVs in groups are approved pursuant to 10 CFR 50.55a(f)(4)(iv) provided you meet all related requirements of OM-10 which include Sections 4.2.2.1 and 4.2.2.3.

Valve relief request VR-49-R3 is granted to leak test the RHR system pressure isolation valves E1150F009 and E1150F608 as a pair pursuant to 10 CFR 50.55a(f)(6)(i) based on the impracticality of performing testing in accordance with the Code requirements, and in consideration of the burden on the licensee if the Code requirements were imposed on the facility.

Valve relief request VR-51 to stroke time test the diesel generator air start solenoid valves based on the start time of the diesel is granted pursuant to 10 CFR 50.55a(f)(6)(i) for an interim period of 1 year. During the interim period, you must develop acceptance criteria to verify that these valves adequately stroke during the monthly diesel test which would satisfy the Code quarterly test frequency requirements.

May 2, 1996

If you have any questions regarding the enclosed safety evaluation, please contact me at (301) 415-1341.

Sincerely,

Original Signed By:

Mark Reinhart, Acting Director
Project Directorate III-1
Division of Reactor Projects - III/IV
Office of Nuclear Reactor Regulation

Enclosure: Safety Evaluation

Docket No. 50-341

cc w/encl: See next page

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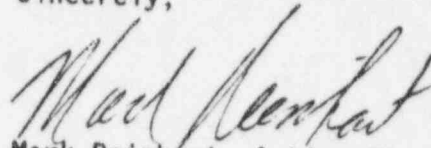
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If you have any questions regarding the enclosed safety evaluation, please contact me at (301) 415-1341.

Sincerely,



Mark Reinhart, Acting Director
Project Directorate III-1
Division of Reactor Projects - III/IV
Office of Nuclear Reactor Regulation

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cc w/encl: See next page