

LICENSEE EVENT REPORT (LER)

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|---|--------------------------------------|----------------------|
| FACILITY NAME (1) Susquehanna Steam Electric Station - Unit 2. | DOCKET NUMBER (2) 0 5 0 0 0 3 8 8 | PAGE (3) 1 OF 0 1 |
|---|--------------------------------------|----------------------|

TITLE (4)
Unanticipated ESF Actuation - RHR Shutdown Cooling.

| EVENT DATE (5) | | | LER NUMBER (6) | | | REPORT DATE (7) | | | OTHER FACILITIES INVOLVED (8) | | |
|-------------------------------|-----|------|----------------|-------------------|-----------------|-----------------|-----|------|-------------------------------|--|------------------|
| MONTH | DAY | YEAR | YEAR | SEQUENTIAL NUMBER | REVISION NUMBER | MONTH | DAY | YEAR | FACILITY NAMES | | DOCKET NUMBER(S) |
| 0 1 | 2 9 | 8 5 | 8 5 | 0 0 7 | 0 0 | 0 2 | 2 8 | 8 5 | | | 0 5 0 0 0 |
| DOCKET NUMBER(S) 0 5 0 0 0 | | | | | | | | | | | |

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|---------------------------|--|-------------------------------------|----------------------|--------------------------|--|--------------------------|--|--|--|--|
| OPERATING MODE (9) 3 | THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11) | | | | | | | | | |
| POWER LEVEL (10) 0 0 0 | 20.402(h) | <input checked="" type="checkbox"/> | 50.73(a)(2)(iv) | <input type="checkbox"/> | 73.71(b) | <input type="checkbox"/> | | | | |
| | 20.405(a)(1)(i) | <input type="checkbox"/> | 50.73(a)(2)(v) | <input type="checkbox"/> | 73.71(c) | <input type="checkbox"/> | | | | |
| | 20.405(a)(1)(ii) | <input type="checkbox"/> | 50.73(a)(2)(vii) | <input type="checkbox"/> | OTHER (Specify in Abstract below and in Text, NRC Form 366A) | | | | | |
| | 20.405(a)(1)(iii) | <input type="checkbox"/> | 50.73(a)(2)(viii)(A) | <input type="checkbox"/> | | | | | | |
| | 20.405(a)(1)(iv) | <input type="checkbox"/> | 50.73(a)(2)(viii)(B) | <input type="checkbox"/> | | | | | | |
| | 20.405(a)(1)(v) | <input type="checkbox"/> | 50.73(a)(2)(ix) | <input type="checkbox"/> | | | | | | |

LICENSEE CONTACT FOR THIS LER (12)

| | |
|--|--|
| NAME L.A. Kuczynski - Nuclear Plant Specialist, III | TELEPHONE NUMBER AREA CODE 7 1 1 7 5 4 2 1 - 1 3 1 7 1 5 1 9 |
|--|--|

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

| CAUSE | SYSTEM | COMPONENT | MANUFACTURER | REPORTABLE TO NPRDS | CAUSE | SYSTEM | COMPONENT | MANUFACTURER | REPORTABLE TO NPRDS |
|-------|--------|-----------|--------------|---------------------|-------|--------|-----------|--------------|---------------------|
| X | BIO | IPIS | BIO | 8 1 N | | | | | |

SUPPLEMENTAL REPORT EXPECTED (14)

| | | | | | |
|--|--|-------------------------------|-------|-----|------|
| <input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE) | <input checked="" type="checkbox"/> NO | EXPECTED SUBMISSION DATE (15) | MONTH | DAY | YEAR |
|--|--|-------------------------------|-------|-----|------|

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single space typewritten lines) (16)

On January 29, 1985, with the Unit in Hot Shutdown, the Residual Heat Removal (RHR) System shutdown cooling suction inboard isolation valve closed unexpectedly. This valve is part of the Primary Containment Isolation System, which is an Engineered Safety Feature. The calibration of the pressure switch, which provides a reactor vessel high pressure permissive signal to the valve, was checked. The switch's 'As Found' setpoint, while low, was within the 'As Found' tolerance band. The switch was recalibrated nevertheless, and shutdown cooling was established. The switch may be replaced pending an evaluation of the system operating restrictions imposed by the use of this type of switch in this application. Additionally, this event will be reviewed by all licensed operators.

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Pennsylvania Power & Light Company

Two North Ninth Street • Allentown, PA 18101 • 215 / 770-5151

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U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

SUSQUEHANNA STEAM ELECTRIC STATION
LICENSEE EVENT REPORT 85-006-00
ER 100450 FILE 841-23
PLAS-046

Docket No. 50-388
License No. NPF-22

Attached is Licensee Event Report 85-006-00. This event was determined reportable per 10CFR50.73(a)(2)(iv), in that the Unit experienced an unanticipated actuation of an Engineered Safety Feature when a containment isolation valve closed.

H.W. Keiser
Superintendent of Plant-Susquehanna

LAK/pjg

cc: Dr. Thomas E. Murley
Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, PA 19406

Mr. R.H. Jacobs
Senior Resident Inspector
U.S. Nuclear Regulatory Commission
P.O. Box 52
Shickshinny, PA 18655

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