PHILADELPHIA ELECTRIC COMPANY

2301 MARKET STREET

P.O. BOX 8699

PHILADELPHIA, PA. 19101

V. S. BOYER SR. VICE PRESIDENT NUCLEAR POWER

December 10, 1984

Docket No. 50-352

Mr. Darrell G. Eisenhut, Director Division of Licensing U.S. Nuclear Regulatory Commission Washington, D. C., 20555

Dear Mr. Eisenhut:

This letter provides follow-up information to my letter dated August 21, 1984 concerning updates on the status of Shift Advisors and Shift SLO personnel participating in the 6 weeks of hot participation experience for the Limerick Generating Station Unit No. 1. In addition, this letter will satisfy the certification requirements of Attachment 3 to Facility Operating License No. NPF-27.

Enclosure 2 of the August 21 letter indicated that written and oral examinations administered to the shift advisors, including test results, would be forwarded when available. Enclosure 1 of this letter includes a copy of the written examination and the oral examination format administered on Sept 21, 1984 for the shift advisors. The oral examination included simulator scenarios and a Plant Control Room walkaround. These oral exams were administered and graded by the LGS Assistant Station Superintendent and the Operations Engineer. The written exam was a 4-hour exam covering plant systems, procedures, and technical specifications and was administered and graded by the LGS Training Coordinator.

8412140102 841210 PDR ADOCK 05000352 V PDR The examination grades for the two Shift Advisor candidates are as follows:

I. ORAL EXAMINATION

	Simulator	Control Room Walkaround	
D. H. Sparks	PASS	PASS	
F. L. Shanaman	PASS	PASS	

II. WRITTEN EXAMINATION

			Section	Section II	Section III	Overall
D.	н.	Sparks	77.4	84.5	84.5	82.2
F.	L.	Shanaman	92.8	83.3	78.9	84.9

Records for this Shift Advisor Training are appropriately documented and kept by the LGS-Nuclear Training Section.

Based upon the foregoing it is hereby certified, in accordance with Attachment 3 to Facility Operating License NPF-27, that D.H. Sparks and F. L. Shanaman are the special assigned advisors and they have been examined and have been determined to be competent to provide advice to the operating shifts.

Enclosure 2 lists SLO personnel who participated in the Hot Participation Experience Training (>20% Rx power) for 6 weeks in order to supplement their cold license due to having no previous "hot" experience. This list does not include Mr. Charles P. Gillespie who is the only Shift Superintendent (SLO) designated to have a Shift Advisor on Shift with him at all times when not in Cold Shutdown condition. Mr. Gillespie did, however, participate in the 6-week program to help improve his "hot plant" awareness.

Records indicating appropriate documentation of the six weeks of hot participation experience training for each SLO participant are kept by the LGS-Nuclear Training Section.

If you have any questions or require additional information, please do not hesitate to contact us.

Very truly yours,

V. S. Boyen

Attachments

cc: Dr. T. E. Murley, Administrator See Attached Service List cc: Judge Helen F. Hoyt Judge Jerry Harbour Judge Richard F. Cole Judge Christine N. Kohl Judge Gary J. Edles Judge Reginald L. Gotchy Troy B. Conner, Jr., Esq. Ann P. Hodgdon, Esq. Mr. Frank R. Romano Mr. Robert L. Anthony Ms. Phyllis Zitzer Charles W. Elliott, Esq. Zori G. Ferkin, Esq. Mr. Thomas Gerusky Director, Penna. Emergency Management Agency Angus Love, Esq. David Wersan, Esq. Robert J. Sugarman, Esq. Martha W. Bush, Esq. Spence W. Perry, Esq. Jay M. Gutierrez, Esq. Atomic Safety & Licensing Appeal Board Atomic Safety & Licensing Board Panel Docket & Service Section (3 copies) James Wiggins Timothy R. S. Campbell

COMMONWEALTH OF PENNSYLVANIA :

SS.

COUNTY OF PHILADELPHIA

V. S. Boyer, being first duly sworn, deposes and says:

:

That he is Senior Vice President of

Philadelphia Electric Company; that he has read the foregoing

letter with respect to shift staffing for Limerick Generating

Station Unit No. 1 and knows the contents thereof; and that the

statements and matters set forth therein are true and correct to

the best of his knowledge, information and belief.

V. S. Boyen

Subscribed and sworn to before me this 10⁷⁴ day of December. 1984

Notary Public

JUDITH Y. FRANKLIN Notary Public, Phila., Phila. Co. My Commission Expires July 28, 1987

Enclosure 1 SHIFT ADVISOR CURRENTLY A LICENSED OF MATOR HEART **EXAMINATION REPORT** YES NO G APPLICANT'S HAME BEACTOR LOCATION WRITTEN EXAMINATION ADMINISTERED BY: DATE GRADED BY: SHIFT GRADE EVALUATION ADVISOR PASSED -CATEGORY GRADES * 2 * 3 FAILED @ ORAL EXAMINATION ADMINISTERED BY: DATE SHIFT PASSED -ADVISOR FAILED -SIMULATOR EXAMINATION ADMINISTERED BY: DATE PASSED -SHIFT ADVISOR FAILED -COMMENTS MARGINAL -PASS -FAIL -RECOMMENDATION

Published State (1997)	Enclosure 1
COMMENT'S:	
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Enclosure 1 SYSTEMS CONTROL ROOM (Major, Auxiliary and Engineered Safeguards Systems) 2.0 EQUIPMENT 2.1 Purpose 2.2 Flow Path 2.3 Normal Parameters 2.4 Components 2.5 System Behavior and Response 3.0 INSTRUMENTATION 3.1 Detector 3.2 Malfunction 3.3 Control Room Indication 4.0 REACTOR PROTECTION 4.1 Alarms/Setpoints 4.2 Safety System Input 4.3 Interlocks 5.0 PROCEDURES 5.1 Normal Procedures Abnormal Procedures 5.2 5.3 Emergency Procedures 6.0 REACTIVITY EFFECTS 7.0 ADMINISTRATIVE REQUIREMENTS 7.1 Technical Specifications 7.2 Facility Requirements COMMENTS: (Required for "U")

В.	CONTROL ROOM (Nuclear and Redistion Instruments)	Jan State of the s			
3.0	INSTRUMENTS	/A / B / C / D / E			
	3.1 Detectors				
	3.2 Malfunctions				
	3.3 Control Room Indications				
	3.4 Channel Components				
	3.5 Compensation/Discriminator				
	3.6 Input to Control System				
4.0	REACTOR PROTECTION				
	4.1 Alarms/Setpoints				
	4.2 Safety System Input				
	4.3 Interlocks				
5.0	PROCEDURES				
	5.1 Normal Procedures				
	5.2 Abnormal Procedures				
	5.3 Emergency Procedures				
7.0	ADMINISTRATIVE REQUIREMENTS				
	7.1 Technical Specifications				
	7.2 Facility Requirements				
COM	MENTS: (Required for "U")				

Enclosure 1 CONTROL ROOM (Electrical) 2.0 EQUIPMENT 2.1 Purpose 2.2 Flow Path 2.3 Normal Parameters 2.4 Components 2.5 System Behavior or Response 3.0 INSTRUMENTS 3.2 Interlocks 3.4 Control Room Indication 5.0 PROCEDURES 5.1 Normal Procedures 5.2 Abnormal Procedures 5.3 Emergency Procedures 7.0 ADMINISTRATIVE REQUIREMENTS 7.1 Technical Specifications 7.2 Facility Requirements COMMENTS: (Required for "U")

Applicant's Signature

LIMERICK GENERATING STATION Shift Advisor

			Facility: Limer Reactor Type: Date Administered: Examiner: Applicant:	BW	/R	
NSTRUCTIO	NS TO APPLIC	ANT:				
heet on top	of the answer s question. Exam	heets. Points for	enswers on one side each question are i all be picked up fo	ndicate	d in parentheses	
			% of			
Category Value	% of Total	Applicant's Score	Category Value	Car	tegory	
18	33			1.	Plant Systems	
18	33			2.	Procedures - Normal Emergency	and
19	34			3.	Administrative Controls Technical Specifications	and
55	100				TOTALS	
		Final Grade	%			
All work don	ne on this exam	is on my own, I hav	e neither given nor	receive	ed aid.	

SECTION 1: PLANT SYSTEMS

- (1.0) 1.1 What precautions should be observed prior to draining the Scram Discharge Volume after a scram?
- (1.0) 1.2 What systems/requirements would you advise the shift superintendent as being required prior to start-up of a recirculation pump? (Assume plant in Cold Shutdown and plant start-up COL in progress in accordance with GP-2).
- (.75) 1.3 a. Why are drywell and suppression chamber inerted?
- (.75) b. What are the consequences to the plant if the suppression chamber to drywell vacuum breakers do not properly operate during a LOCA?
- (2.0) 1.4 Explain the plant response in the event the on-line EHC pressure regulator fails to high output. (Assume plant operating at 100% Rx power.)
- (1.5) 1.5 Following the automatic initiation of the "A" Standby Liquid Control Pump, the "A" explosive valve fails to open. Explain how this would or would not hamper SLCS operation.
- (2.0) 1.6 List four modes of the RHR system and briefly discuss the flowpath of each one from discharge of RHR pump to return.
- (2.0) 1.7 Certain reactor fuels require fuel preconditioning, while others do not. Is preconditioning necessary or unnecessary at the LGS? Explain your choice of answers.
- (2.0) 1.8 What signals are required to automatically initiate the following systems:
 - a. LPCI
 - b. Emergency Diesel Generators
 - c. ADS
 - d. HPCI
- (2.0) 1.9 Assume the following initial conditions:
 - 100% power
 - Feedwater in 3-element control

One steam flow transmitter's output signal fails <u>low</u>. Assuming no operator action, how will this failure effect RFPT operation and what will be the final reactor level?

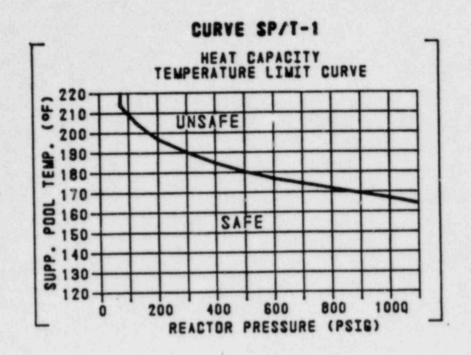
1.10 With regard to the Redundant Reactivity Control System:

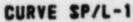
- (2.0) a. What will cause on Alternate Rod Insertion?
- (1.0) b. How will operation of the ARI valves effect speed of control rod insertion on scram?

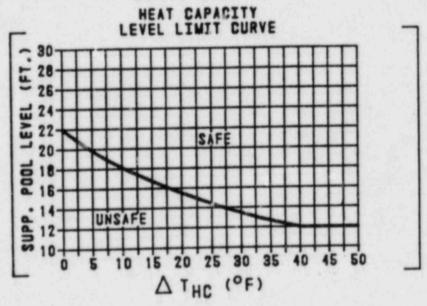
SECTION 2: PROCEDURES - NORMAL AND EMERGENCY

	2.1	List the entry conditions for the following:
(1.0)		a. T-101, Reactor Control
(1.0)		b. T-102, Containment Control
(1.5)	2.2	A fire occurs in the Control Room which requires evacuation and plans shutdown at the Remote Shutdown Panel. As a shift advisor, what immediate operator actions would you advise the Shift Superintendent to check for completion prior to control evacuation as per SE-1 (Plans Shutdown from Outside Control Room)?
(1.5)	2.3	As a shift advisor you are making rounds of the control panels with your Shift Superintendent (SST) after shift relief. The ACO informs the SST that he suspects a failed jet pump due to an unexplained decrease in core flow indication. What other symptoms might you suggest the SST check on prior to confirming a failure of a jet pump in accordance with ON-100? (Three others required).
(2.0)	2.4	Briefly discuss the purpose behind use of GP-18 (Scram Review Procedure) checkoff list after automatic or manual scram condition.
(2.0)	2.5	As per GP-4, Rapid Plant Shutdown, guidance is provided to take the reactor from at power condition to hot shutdown condition. As a shift advisor, what three procedural steps would you recommend to the SST to check on being completed to ensure an orderly progression to hot shutdown?
(1.0)	2.6	GP-2, Plant Startup, states that with EHC pressure setpoint at 150 psig, werify proper operation of the turbine bypass valves and then raise the pressure setpoint to 600 psig. Other than to close the BPV's, why is the setpoint increased and held at 600 psig?
(2.0)	2.7	Given the following conditions, use the attached graphs from T-102, Containment Control, to determine the minimum suppression pool level required to safely absorb an automatic vessel depressurization:
		Reactor pressure = 650 psig Suppression pool temperature = 160°F
(2.0)	2.8	As a shift advisor, what symptoms would indicate to you that a SRV had, in fact, inadvertantly opened as per OT-114 (four required).
(2.0)	2.9	As a shift advisor observing a reactor startup from hot shutdown on Unit 1, the SST cautions the ACO to ensure he watches closely for any high notch rod worths which may occur. What conditions might the reactor exhibit high rod notch worths and how would it be possible to get into these situations?

(2.0) 2.10 As a shift advisor, under what types of plant conditions might you recommend a rapid plant depressurization (via manual ADS initiation) to the Shift Superintendent?







SECTION 3: ADMINISTRATIVE CONTROLS AND TECHNICAL SPECIFICATIONS

- 3.1 Describe the actions that must be taken if the following occur during a reactor startup:
- (1.0) a. RWM fails
- (1.0) b. RSCS fails
- (2.0) 3.2 What topics are required in the verbal report that takes place at shift turnover?
 - 3.3 During operation at 100% Rx power, the ACO on Unit 1 informs the SST that the process computer indicates that MAPRAT is equal to 1.01. Indicate your recommendations to the SST in the following areas:
- (1.0) a. Is a thermal limit being exceeded? Explain.
- (1.0) b. What is the thermal limit consideration associated with MAPRAT.
- (3.0) 3.4 The attached portion of Technicial Specifications should be used to answer the following question:

Scenario: Plant at 100% Rx power. The D-12 has been out of service due to a faulty blower for three days. Estimated time of repair is five days. About two hours into the shift the P.O. reports to control room that the "C" ESW pump has shutdown during a surveillance test due to motor overcurrent and he has noticed the smell of light electrical smoke in the Spray Pond Pump House.

As the Shift Advisor, what would be your recommendation to the Shift Superintendent in how long plant operation could continue?

- (2.0) 3.5 With the plant operations at 100% power, the oncoming shift arrives for relief of the 3:30-11:30 p.m. shift. The relieving shift consists of the following:
 - a. Shift Superintendent (SLO) with accompanying Shift Advisor
 - b. Shift Supervisor (SLO) absent
 - c. Control Operator (LO)
 - d. Unit 1 Assistant Control Operator (LO) absent
 - e. Plant Operator absent
 - f. Assistant Plant Operator
 - g. Inside Auxiliary Operator
 - h. Outside Auxiliary Operator

i. Helpers (2)

The operating shift being relieved has the following compliment with each just completing a double shift because of a major winter storm:

- a. Shift Superintendent (SLO)
- b. Shift Supervisor (SLO)
- c. Control Operator (LO)
- d. Unit 1 Assistant Control Operator (LO)
- e. Plant Operator
- f. Assistant Plant Operator
- g. Inside Auxiliary Operator
- h. Outside Auxiliary Operator
- i. Helpers (2)

By use of the attached Technical Specification state what your recommendation to the on-coming Shift Superintendent would be concerning maintaining continued, safe plant operations.

(3.0) 3.6 The following question should be answered utilizing the attached portion of Technical Specifications:

Scenario: With the reactor operating at 45% power during power ascension tests following a "rod swap" (as per Rx Engineer Instructions) and the completion of control rod scram testing to satisfy Tech. Spec. surveillance test 4.1.3.2.C., it is determined that the average scram insertion time to notch 39 of all operable control rods is .87 seconds.

The Shift Superintendent, after receiving this information, takes action in accordance with LGS Technical Specifications. As a Shift Advisor, what action should be take? Why?

- (1.0) 3.7 a. What is the relationship between a Safety Limit (SL) and a Limiting Safety System Setting (LSSS)?
- (1.0) b. What are the LGS Safety Limits?
- (1.0) C. What actions would you expect the Shift Superintendent to take in the event a safety limit is exceeded?
- (1.0) 3.8 Under what plant conditions could you recommend a manual plant shutdown to the Shift Superintendent?
- (1.0) 3.9 As a Shift Advisor just relieving the shift you are making a check of the ACO turnover sheet (attached). What might you recommend to the Shift Superintendent as a result of this review?

1. R 2. a	oncoming A Condis Start Y Off-going A Contrella Oncoming A Condis Startey ON MODE RUN GMWE 1105 CMWT 3297 On Restrictions on Reactor Parameters: U/1 100% U/2 N/A On Remarks: Conducting Worranty Plum for another 48 hours.
1. R	X MODE RUN GMWE 1105 CMWT 3297 Restrictions on Reactor Parameters: U/1 100% U/2 N/A
2. a	. Restrictions on Reactor Parameters: U/1 100% U/2 N/A
ь	40. 3 mg/g/g/m/c a 1,5 mg/g/m/c a 1
	. Remarks: Conducting Worranty Run for another 48 hours.
3. P	
	Primary Containment Established: U/1 Yes J/2 N/A
4. S	Secondary Containment Established: U/1 Yes U/2 N/A
5. <u>s</u>	SURVEILLANCE LOG OUT-OF-SPECIFICATION READINGS (Explain):
	NSTRUMENTATION BYPASSED OR OUT OF SPECIFICATION-S/U & RUN ODE ONLY-(Explain):
	RBM A bypassed - nulling sequence problem; being invistigated by Ite troks.
	The services product production of the treks.
7. S	SYSTEM OR COMPONENT WHICH IS INOP. AS PERMITTED BY TECH. SPEC's:
Ī	tem Reason Date/Time Inop. Return Deadline
1	HPCI AUX.0.1 2/1/85/1600 2/16/85/2400
	limmy I Chara
8. L	og Book Reviewed: RJC (Offgoing Aco) RLL (Oncoming Aco
	og Book Reviewed: RJC (Offgoing Aco) RU (Oncoming Aco
9. W	alkaround Completed for Control Panels: PJC (Offgoing Aco)
9. W	Control Panels Required Line-up Verified: QC (Offgoing ACO)
9. W	Control Panels Required Line-up Verified: (Oncoming Aco) (Oncoming Aco) (Offgoing Aco) (Oncoming Aco) (Oncoming Aco) (Oncoming Aco)

ENCLOSURE 2

Senior Licensed Operators at LGS Unit 1 who participated in a minimum 6-week Hot Participation Training (>20% Rx power) program at Peach Bottom Atomic Power Station:

Name	Position	Dates Participated
Collins, Greg	Shift Supervisor	08/06/84 thru 09/28/84
Crosier, Thomas V.	Shift Supervisor	08/28/84 thru 10/09/84
Paton, Glenn A.	Shift Supervisor	08/06/84 thru 10/03/84
Romano, Anthony	Shift Supervisor	08/28/84 thru 10/09/84
Russell, William N.	Shift Supervisor	09/03/84 thru 10/12/84
Stanley, William	Shift Supervisor	08/06/84 thru 09/20/84
Tindall, Ronald	Shift Supervisor	08/06/84 thru 09/19/84

Note: Charles P. Gillespie (Shift Superintendent) also attended 6 weeks of training, 08/06/84 thru 09/24/84, but he is the lone Shift Superintendent who has a Shift Advisor on his Shift.