

ENCLOSURE

DESCRIPTION OF VIOLATIONS

Jersey Central Power and Light Company  
Morristown, New Jersey 07960  
Docket No. 50-219  
License No. DPR-16

Certain activities under your license appear to be in violation of AEC regulations and license requirements. These apparent violations are considered to be of Category II severity.

1. Directorate of Licensing letter to Jersey Central Power and Light Company dated January 30, 1974, temporarily supersedes and refers to containment system operating status. Paragraph B.5. of that letter stated that "Reactor Operations may continue provided that not more than 25% of the vacuum breakers are inoperable.

Contrary to this requirement, the drywell to suppression chamber vacuum breakers failed to meet operable limits during surveillance testing:

- a. Breakers V-26-5, 6, 9 and 11 failed to demonstrate operability (JCP&L letter to D.L. dated February 25, 1974).
  - b. Breakers V-26-4, 5, 6 and 12 failed to demonstrate operability (JCP&L letter to D.L. dated March 15, 1974).
2. Technical Specification 3.5 refers to containment system operating status. Section 3.5.A.1 states in part that "Primary containment integrity shall be maintained at all times when the reactor is critical or when the reactor temperature is above 212°F and fuel is in the reactor vessel...".

Contrary to these requirements for containment integrity,

- a. Closure of valves V-16-1 and V-16-14 in the reactor coolant cleanup system did not permit isolation of the system while the reactor was critical (JCP&L letter to D.L. dated February 27, 1974).
  - b. Two torus to drywell vacuum breaker valves were open for maintenance with reactor temperature above 212°F (JCP&L letter to D.L. dated March 18, 1974).
3. Technical Specification 3.7 refers to operating status of the auxiliary electrical power supply. Section 3.7.B states in part that "The reactor may remain in operation for a period not to exceed 7 days in any 30 day period if a startup transformer is out of service".

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DATE▶							

Contrary to these requirements the SB startup transformer was unavailable for service for a time period greater than 7 days during a 30 day period (JCP&L letter to D.L., dated March 15, 1974).

4. Technical Specifications 4.5 refers to containment system leakage. Section 4.5.F.1.d states in part "If the total leakage rates... are exceeded, repairs and retests shall be performed...".

Contrary to the above, the total allowable leakage rates were exceeded by main steam isolation valves NS04A and NS04B (JCP&L letter to D.L. dated March 18, 1974).

5. Technical Specifications 3.1 refers to status of plant instrumentation which performs a protective function. Table 3.1.1.B.2 specifies that the main steam line high flow sensors actuate at a differential pressure of  $< 120\%$  of rated steam flow.

Contrary to the above, one sensor was bypassed and inoperable (JCP&L letter to D.L. dated March 22, 1974).

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DATE▶					

To:

James P. O'Reilly  
Directorate of Regulatory Operations  
Region I  
631 Park Avenue  
King of Prussia, Pennsylvania 19406

From:

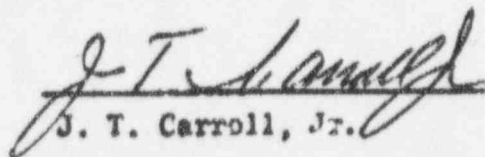
Jersey Central Power & Light Company  
Oyster Creek Nuclear Generating Station Docket #50-219  
Forked River, New Jersey 08731

Subject:

Abnormal Occurrence Report No. 50-219/74/ 27

The following is a preliminary report being submitted  
in compliance with the Technical Specifications  
paragraph 6.6.2.

Preliminary Approval:

  
J. T. Carroll, Jr.      4/18/74  
Date

cc: Mr. A. Giambusso

~~8344119411~~ (388)

B/571

Initial Telephone  
Report Date: 4/18/74

Date of  
Occurrence: 4/17/74

Initial Written  
Report Date: 4/18/74

Time of  
Discovery: 1500

OYSTER CREEK NUCLEAR GENERATING STATION  
FORKED RIVER, NEW JERSEY 08731

Abnormal Occurrence  
Report No. 50-219/74/27

IDENTIFICATION  
OF OCCURRENCE:

Violation of the Technical Specifications, paragraph N/A,  
Failure of three hydraulic shock and sway arrestors (two  
located on the main steam line and one on the feedwater line).

This event is considered to be an abnormal occurrence as de-  
fined in the Technical Specifications, paragraph 1.15D.

CONDITIONS PRIOR  
TO OCCURRENCE:

<input type="checkbox"/> Steady State Power	<input type="checkbox"/> Routine Shutdown
<input type="checkbox"/> Hot Standby	<input type="checkbox"/> Operation
<input type="checkbox"/> Cold Shutdown	<input type="checkbox"/> Load Changes During
<input checked="" type="checkbox"/> Refueling Shutdown	<input type="checkbox"/> Routine Power Operation
<input type="checkbox"/> Routine Startup	<input type="checkbox"/> Other (Specify)
<input type="checkbox"/> Operation	

The plant was shutdown with reactor coolant <212°F and  
in the process of preparing for refueling operations.

DESCRIPTION  
OF OCCURRENCE:

On April 17, 1974, an inspection was conducted of all Bergen  
Paterson hydraulic shock and sway arrestor units installed in  
the drywell. As a result of this inspection, three units, re-  
built in September 1973 with molded polyurethane material,  
were found inoperable as determined by the absence of fluid  
level indication in the accumulators. The inoperable units  
were as follows:

<u>Serial #</u>	<u>System</u>	<u>Elevation</u>
487530	South Main Steam	23'
487512	South Main Steam	51'
487519	South Feedwater	51'

APPARENT CAUSE  
OF OCCURRENCE:

- |                                        |                                                    |
|----------------------------------------|----------------------------------------------------|
| <input type="checkbox"/> Design        | <input type="checkbox"/> Procedure                 |
| <input type="checkbox"/> Manufacture   | <input type="checkbox"/> Unusual Service Condition |
| <input type="checkbox"/> Installation/ | <input type="checkbox"/> Inc. Environmental        |
| <input type="checkbox"/> Construction  | <input type="checkbox"/> Component Failure         |
| <input type="checkbox"/> Operator      | <input type="checkbox"/> Other (Specify)           |

None of the units have as yet been disassembled and, consequently, the failure mechanism is unknown at this time.

ANALYSIS OF  
OCCURRENCE:

Had the design seismic event occurred during power operation, the restraining capabilities of these units may have been seriously impaired and, consequently, degraded the structural integrity of the steam and feedwater lines in question.

CORRECTIVE  
ACTION:

Current plans are to replace these units and others in the primary containment which still contain molded polyurethane material with units rebuilt exclusively with ethylene propylene material.

FAILURE DATA:

Manufacturer: Bergen-Paterson  
Type: HSSA-10  
Serial Nos.: 487530  
487512  
487519

Prepared by:

DR Reeves, Jr

Date:

4/18/74