



Wayne H. Jens  
Vice President  
Nuclear Operations

Fermi-2  
6400 North Dixie Highway  
Newport, Michigan 48166  
(313) 586-4150

March 6, 1985  
NE-85-0343

Director of Nuclear Reactor Regulation  
Attention: Mr. B. J. Youngblood, Chief  
Licensing Branch No. 1  
Division of Licensing  
U. S. Nuclear Regulatory Commission  
Washington, D. C. 20555

Dear Mr. Youngblood:

Reference: Fermi 2  
NRC Docket No. 50-341

Subject: Suppression Pool Temperature Measurements

The purpose of this letter is to respond to a recent request from the NRR Project Manager (Mr. M. D. Lynch). The discussion below describes the Detroit Edison program that will be used to measure suppression pool temperatures during startup testing and initial plant operation.

Prior to operation at full power, a test shall be performed involving a safety/relief valve (SRV) actuation to accomplish the following:

- o Measure the pool temperatures in all eight sectors of the suppression pool at approximately 5 minute intervals.
- o Demonstrate that all adjacent temperature readings are within 45°F of each other throughout SRV actuation with suppression pool mixing due to operation of a reactor heat removal system pump.

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- o Reperform an SRV actuation without suppression pool mixing or cooling and record the test data in the manner described above. (Technical Specification maximum temperature limitations will not be exceeded.) Measure temperatures in all eight sectors of the suppression pool for 48 hours or until thermal equilibrium of the pool is reached. Demonstrate that using the seven sector method, excluding the highest sector suppression pool temperature reading, will yield a conservative value for the bulk temperature of the suppression pool within a 48-hour period.

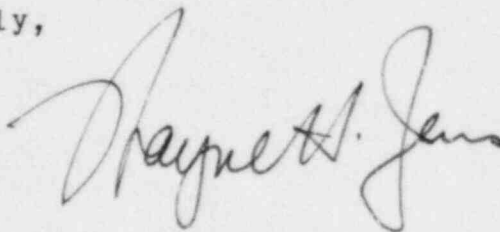
During the first year of normal operation under Operational Conditions 1 and 2, the following information shall be recorded:

- o The suppression pool temperatures in all eight sectors every 24 hours.
- o The conditions in the suppression pool which at the time promoted mixing, if applicable (e.g., cooling of the suppression pool or actuation of an SRV).

The data recorded above shall be submitted to the NRC before startup following the first refueling outage. In the event the 45°F temperature differential is exceeded, such an occurrence shall be reported per 10CFR50.73.

Please direct any questions to Mr. O. K. Earle at (313) 586-4211.

Sincerely,



cc: Mr. P. M. Byron  
Mr. M. D. Lynch  
USNRC Document Control Desk  
Washington, D. C. 20555