

Docket File  
50-454/456



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

May 3, 1996

Mr. D. L. Farrar  
Manager, Nuclear Regulatory Services  
Commonwealth Edison Company  
Executive Towers West III  
1400 Opus Place, Suite 500  
Downers Grove, IL 60515

SUBJECT: SAFETY EVALUATION OF INSERVICE INSPECTION PROGRAM RELIEF REQUEST FOR INSPECTION OF THE PRESSURE SURGE NOZZLE-TO-VESSEL WELD; BYRON AND BRAIDWOOD STATIONS (TAC NOS. M95088, M95089, M95172, AND M95173)

Dear Mr. Farrar:

By letter dated March 28, 1996, Commonwealth Edison Company (ComEd, the licensee), submitted requests for relief (Relief Requests NR-19 and NR-24) from the American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code (Code), Section XI, requirements for the Byron Station, Units 1 and 2 and Braidwood Station, Units 1 and 2, First Ten Year Interval Inservice Inspection (ISI) Program. Alternatives were proposed for the examination of the pressurizer nozzle-to-vessel weld and the nozzle inner radius on the basis that compliance with the Code requirement would result in high radiation exposure to the workers and unusual difficulty without a compensating increase in the level of quality and safety. Additional information was provided in ComEd's letter of April 23, 1996.

The Code of Federal Regulations, 10 CFR 50.55a, requires that the ISI of certain ASME Code Class 1, 2, and 3 components be performed in accordance with Section XI of the applicable edition and addenda of the ASME Code, except where specific written relief has been requested by the licensee and granted by the Commission pursuant to 10 CFR 50.55a, paragraph (g)(6)(i) or alternatives to the Code authority pursuant to paragraph (a)(3)(i) or (a)(3)(ii).

The staff, with the assistance from our contractor, the Idaho National Engineering Laboratory, reviewed and evaluated the information provided in the submittals. Based on the information submitted, the staff adopts the contractor's conclusions and recommendations presented in their Technical Letter Report attached to the enclosed Safety Evaluation (SE). The alternatives proposed by ComEd in Requests for Relief Nos. NR-19 and NR-24 for

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Byron and Braidwood, respectively, are hereby authorized pursuant to 10 CFR 50.55a (3)(a)(ii). However, if the insulation is removed in the future for any reason, the licensee shall conduct a volumetric examination of the nozzle-to-vessel weld and nozzle inside radius section.

Sincerely,

Original signed by:  
George F. Dick for:

Robert A. Capra, Director  
Project Directorate III-2  
Division of Reactor Projects III/IV  
Office of Nuclear Reactor Regulation

Docket Nos. STN 50-454, STN 50-455,  
STN 50-456, AND STN 50-457

Enclosure: Safety Evaluation w/attch

cc w/encl: See next page

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