

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Indian Point Unit 3	DOCKET NUMBER (2) 0 5 0 0 0 2 8 5	PAGE (3) 1 OF 02
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TITLE (4)
Unit Trip Signal Received While Subcritical

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
1	0	1	8	4	0	1	0	1			0 5 0 0 0
1	0	1	8	4	0	1	0	1			0 5 0 0 0

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)

OPERATING MODE (9) N	20.402(b)	20.406(e)	<input checked="" type="checkbox"/>	80.73(a)(2)(iv)	73.71(b)
POWER LEVEL (10) 0 0 1 0	20.406(a)(1)(i)	80.38(a)(1)	<input type="checkbox"/>	80.73(a)(2)(v)	73.71(a)
	20.406(a)(1)(ii)	80.38(a)(2)	<input type="checkbox"/>	80.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
20.406(a)(1)(iii)	80.73(a)(2)(i)	<input type="checkbox"/>	80.73(a)(2)(viii)(A)		
20.406(a)(1)(iv)	80.73(a)(2)(ii)	<input type="checkbox"/>	80.73(a)(2)(viii)(B)		
20.406(a)(1)(v)	80.73(a)(2)(iii)	<input type="checkbox"/>	80.73(a)(2)(ix)		

LICENSEE CONTACT FOR THIS LER (12)

NAME Floyd W. Gumble	TELEPHONE NUMBER AREA CODE: 9 1 4 7 3 9 - 8 2 0 0
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COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS
X	I	G F U	B 5 6 9	Y					

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE) NO

EXPECTED SUBMISSION DATE (15)

MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On October 13, 1984, with the reactor subcritical at hot zero power (HZP), a trip signal was initiated at 0136 hours, as the result of a blown control power fuse on Channel 35 Intermediate Range. The plant had recently been brought to HZP as part of a controlled shutdown for a planned maintenance/inspection outage. The trip signal resulted in the automatic insertion of the Shutdown and Control Rod Banks, which were being gradually inserted as part of the shutdown/cooldown procedures.

The fuse (Bussman, 5 amp, slow blow) on Intermediate Range 35 was replaced by 0143 hours.

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PDR ADOCK 05000286
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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1) Indian Point Unit 3	DOCKET NUMBER (2) 0 5 0 0 0 2 8 6	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
		84	014	00	02	OF 02

TEXT (If more space is required, use additional NRC Form 365A's) (17)

On October 13, 1984, the plant was subcritical in the Hot Zero Power condition, in the process of being brought to cold shutdown for a scheduled maintenance/inspection outage. As the plant was passing the P-6 setpoint during power descent, chattering on the P-6 relay was observed. It is believed that the current surges on the Instrument Bus resultant from this chattering stressed the Channel 35 Intermediate Range control power fuse, which is feed from this bus, causing the fuse to blow and generate a reactor trip signal. The trip signal resulted in the automatic insertion of the Shutdown and Control Rod Banks, which were being gradually inserted as part of the shutdown/cooldown procedures. Additionally, the fuse failure led to a subsequent power surge on the instrument bus. The protective circuitry on 31 Source Range responded to this perturbation by shorting out the high voltage power supply, as designed. As the plant was very close to the P-6 setpoint at the time, the trip was originally erroneously attributed to the loss of the source range high voltage power supply.

The fuse (Bussman, 5 amp, slow blow) on Intermediate Range 35 was replaced by 0143 hours, and the protective circuitry for Channel 31 Source Range was reset.

This event is reportable under 10CFR50.73(a)(2)(iv), which became effective January 1, 1984. A similar event occurred on January 28, 1984 (LER 84-002-00).

It is acknowledged by the New York Power Authority that the four-hour notification of this trip required under 10CFR50.72(b)(2)(ii) was made late. The need for reportability was not recognized by plant personnel at the time for two reasons: first, this type of event was not reportable under the old LER rule, and, secondly, there have been no plant trips at subcritical conditions at Indian Point 3 since the new LER rule came into effect. The omission was discovered during a routine administrative review of watch logs on October 18, 1984, and the four-hour notification was made immediately thereafter. To preclude repetition of this event, all licensed personnel will be reinstructed in all aspects of the LER rule as part of their ongoing retraining program.

Indian Point 3
Nuclear Power Plant
P.O. Box 215
Buchanan, New York 10511
914 739.8200



November 9, 1984
IP-FWG-3777

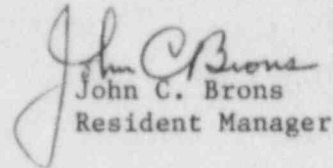
Docket No. 50-286
License No. DPR-64

Document Control Desk
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Dear Sir:

The attached Licensee Event Report LER 84-014-00 is hereby submitted in accordance with the requirements of 10CFR50.73. This event is of the type defined in Paragraph 50.73(a)(2)(iv).

Very truly yours,


John C. Brons
Resident Manager

FWG/bam
Attachment

cc: Dr. Thomas Murley
Regional Administrator
Region 1
U. S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, Pennsylvania 19406

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