

UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

SUPPORTING AMENDMENT NO. 85 TO FACILITY OPERATING LICENSE NO. DPR-40

OMAHA PUBLIC POWER DISTRICT

FORT CALHOUN STATION, UNIT NO. 1

DOCKET NO. 50-285

Introduction:

Following the accident at Three Mile Island, Unit 2, the NRC staff developed NUREG-0660, "NRC Action Plan Developed as a Result of the TMI-2 Accident," to provide a comprehensive and integrated plan to improve safety at nuclear power reactors. Specific NUREG-0660 items that were approved by the Commission for implementation were issued in "Clarification of TMI Action Plan Requirements," NUREG-0737. NUREG-0737 and its Supplement 1 (Generic Letter 82-33, "Requirements for Emergency Response Capability") specified that amended Technical Specifications would be required in order to implement several of the items. Subsequently, on September 20, 1982, NRC issued Generic Letter 82-16, "NUREG-0737 Technical Specifications," requesting all pressurized water reactor licensees to (a) review their existing Technical Specifications against the GL 82-16 guidance and (b) submit proposed Technical Specifications for those items that deviated from the GL 82-16 guidance.

On October 27, 1982, and January 7, 1983, the Omaha Public Power District (OPPD) responded to GL 82-16. The latter submittal to the NRC proposed changes to two sections of the Fort Calhoun Technical Specifications. The submitted changes are responsive to TMI Action Plan Item I.A.1.3. ("Limit Overtime") and II.K.3.3 ("Reporting Safety Valve and Relief Valve Failures and Challenges").

Evaluation

A. Limit Overtime (I.A.1.3)

On June 15, 1982, the staff issued Generic Letter 82-12, which contained a revised version of the Commission's Policy Statement on nuclear power plant staff working hours. Therein, OPPD was requested to revise the administrative section of the Fort Calhoun Station Technical Specifications to adhere to the policy statement guidelines. The objective of the controls is to assure that, to the extent practicable, personnel are not assigned to shift duties while in a fatigued condition that could significantly reduce their mental alertness or their decision making capability. The controls are to be applicable to all plant staff who

8411160470 841011 PDR ADDCK 05000285 PDR perform safety-related functions (e.g., senior reactor operators, reactor operators, auxiliary operators, health physicists, and key maintenance personnel).

Subsequently, OPPD responded on January 7, 1983, by proposing revised Technical Specifications that incorporated administrative procedures for shift overtime. The revised Technical Specifications generally satisfy the intent and requirements of GL 82-12. Furthermore, we have verified that the administrative procedures are consistent with the requirements of the model Technical Specifications. We, therefore, conclude that the proposed Technical Specifications are acceptable and that OPPD has fulfilled the necessary requisites of Item I.A.1.3.

B. Reporting SV and RV Failures and Challenges (II.K.3.3)

In their January 7, 1983 letter, OPPD proposed administrative changes to the Fort Calhoun Station Technical Specifications. These changes addressed Action Item II.K.3.3. According to the submitted changes, NRC will be notified of challenges or failures of safety and power operated relief valves associated with the pressurizer. All challenges and failures will be reported monthly in the licensee monthly report submitted to the Commission.

We conclude that these commitments are responsive to our request. We, therefore, conclude that the Technical Specifications proposed by OPPD related to Item II.K.3.3 are acceptable.

Environmental Consideration

This amendment relates to changes in recordkeeping, reporting or administrative procedures or requirements. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: October 11, 1984

Principal Contributors: D. Powers E. Tourigny