NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
Dresden Nuclear Power Station	0 5 0 0 0 2 4 9	1 OF 0 2

TITLE (4)

Reactor Scram

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
									N/A	0 5 0 0 0
1 0	1 4	8 4	8 4	0 1 6	0 0	1 1	0 7	8 4	N/A	0 5 0 0 0

OPERATING MODE (9)	N	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)			
POWER LEVEL (10)	0 9 0	20.402(b)	20.405(c)	X 50.73(a)(2)(iv)	73.71(b)
		20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
		20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
		20.405(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
		20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
		20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
Lawrence A. Boyle (X-526)	AREA CODE 8 1 5 9 4 2 - 2 9 2 0

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	
X	S H	0 D 2 0	C 6 7 0	N							
X	S H	0 0 2 0	P 3 4 0	N							

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

While performing DOP 4400-8, Circulating Water Flow Reversal, the reactor scrammed due to low condenser vacuum. The reactor scrammed again at 0137 on low vacuum after the vacuum breaker was opened to prevent turbine damage due to low steam seal header pressure because the steam seal valve (S-1) would not open.

Safety significance was minimal since the reactor scram function performed as intended. This is the first occurrence of this type at Dresden.

IE22
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NRC Form 366A
(9-83)

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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TEXT (If more space is required, use additional NRC Form 366A's) (17)

Low condenser vacuum was caused by the 3-4402-A valve not opening completely while performing the Circulating Water Flow Reversal Test, DOP 4400-8. An attempt was made to reverse flow but the low condenser reactor scram actuated before this could be accomplished. After the Circulating Water Flow Reversal evolution was completed, condenser vacuum returned to normal and the initial reactor scram was reset. However, about nine minutes later it was noticed that the turbine had low steam seal header pressure due to the steam seal feed water valve (S-1) not opening. To protect against turbine damage the turbine vacuum breaker was opened causing a second reactor scram on low condenser vacuum. The stem for the 3-4402-A valve was lubricated and the valve was cycled six times. The amps on this valve initially was ten amps and the final reading was six amps. Upon further pursuing this valve problem, the manufacturer was contacted on this subject. Upon his suggestion work requests have been written to check the torque settings and sealed bearings within this valve to further ensure proper preventative maintenance. This inspection program will be accomplished during the next refueling outage. However, since this reactor scram the 3-4402-A valve has gone through other Circulatory Water Flow Reversal evolutions without any problems recurring. The S-1 feedwater valve manual lever was found engaged and was manually disengaged to allow the S-1 valve to operate normally by use of the motor operation. Safety significance was minimal since the reactor scram performed as intended. This is the first occurrence of this type at Dresden.

Commonwealth Edison
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R.R. #1
Morris, Illinois 60450
Telephone 815/942-2920

November 7, 1984

DJS Ltr #84-1266

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Licensee Event Report #84-016-0, Docket #050249 is being submitted as required by Technical Specification 6.6, NUREG 1022 and 10 CFR 50.73 (a)(2)(iv).

D.J. Scott
Station Superintendent
Dresden Nuclear Power Station

DJS/kjl

Enclosure

cc: J.G. Keppler, Regional Administrator, Region III
File/NRC
File/Numerical

IE22
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