

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Dresden Nuclear Power Station	DOCKET NUMBER (2) 0 5 0 0 0 2 4 9 1	PAGE (3) 1 OF 0 2
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TITLE (4)
Unit 3 Reactor Scram

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		
									N/A		
1	0	1	4	8	4	8	4	8	0 5 0 0 0		
									0 5 0 0 0		

OPERATING MODE (8) N	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5. (Check one or more of the following) (11)									
POWER LEVEL (10) 0 0 0	<input type="checkbox"/> 20.402(b)	<input type="checkbox"/> 20.406(a)	<input checked="" type="checkbox"/> 80.73(a)(2)(iv)	<input type="checkbox"/> 73.71(b)						
	<input type="checkbox"/> 20.406(a)(1)(i)	<input type="checkbox"/> 80.36(a)(1)	<input type="checkbox"/> 80.73(a)(2)(v)	<input type="checkbox"/> 73.71(e)						
	<input type="checkbox"/> 20.406(a)(1)(ii)	<input type="checkbox"/> 80.36(c)(2)	<input type="checkbox"/> 80.73(a)(2)(vi)	OTHER (Specify in Abstract below and in Text, NRC Form 365A)						
	<input type="checkbox"/> 20.406(a)(1)(iii)	<input type="checkbox"/> 80.73(a)(2)(i)	<input type="checkbox"/> 80.73(a)(2)(vii)(A)							
	<input type="checkbox"/> 20.406(a)(1)(iv)	<input type="checkbox"/> 80.73(a)(2)(ii)	<input type="checkbox"/> 80.73(a)(2)(vii)(B)							
	<input type="checkbox"/> 20.406(a)(1)(v)	<input type="checkbox"/> 80.73(a)(2)(iii)	<input type="checkbox"/> 80.73(a)(2)(x)							

LICENSEE CONTACT FOR THIS LER (12)		TELEPHONE NUMBER	
NAME Kevin Sykes	(X-523)	AREA CODE 8 1 5	9 4 2 - 2 9 2 0

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)										
CAUSE	SYSTEM	COMPONENT	MANUFAC. TURER	REPORTABLE TO NPROS	CAUSE	SYSTEM	COMPONENT	MANUFAC. TURER	REPORTABLE TO NPROS	
D				N						

SUPPLEMENTAL REPORT EXPECTED (14)			EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)	<input checked="" type="checkbox"/> NO					

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

While confirming the automatic insertion of the control rods from low air pressure (DOS 300-5) the scram discharge volume high level bypass circuit was accidentally broken resulting in a full reactor scram. Jumpers were placed on terminals DD8 and DD11 on control panel 903-5 in accordance with DOS 300-5. As the jumpers were being removed, contact was lost from the wires on the terminals causing relay 590-111B (scram discharge volume high level bypass) to open allowing a full reactor scram. To prevent recurrence this procedure will be deleted upon NRC approval or a procedure change will be incorporated. Last similar occurrence was reported per R.O. 84-4 on Docket #050-249. Minimal safety significance since reactor protection system operated as designed and all control rods were already at 00.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
		8 4	— 0 1 7	— 0 0 0 2	OF	0 2

TEXT (If more space is required, use additional NRC Form 366A (1/77))

DOS 300-5 (Automatic Control Rod Insertion on Low Control Air Pressure) is a quarterly surveillance done to meet requirements of I.E. Bulletin 80-17 until the scram discharge volume modification is complete. While performing DOS 300-5 the scram discharge volume high level bypass circuit was accidentally broken resulting in a full reactor scram. As the jumper was being removed from terminals DD8 and DD11, continuity was lost to relay 590-111B which allowed a full reactor scram. It is felt that the procedure DOS 300-5 did not adequately bring to the attention of the Electrician the consequence of losing continuity on relay 590-111B. It is felt that a precaution should be included in the procedure telling the Electrician of a possibility of a reactor scram if continuity is lost. However, the scram discharge volume modification for Unit 3 has been completed and plans are to pursue with the NRC cancelling surveillance DOS 300-5 due to this SDV modification being completed, which ensures adequate instrument volume hydraulic coupling. Therefore deletion of DOS 300-5 will preclude recurrence. If the surveillance is not cancelled, a better procedure change will be incorporated and a better connector will be installed to allow the jumper to be installed without the fear of losing continuity.

Minimal safety consequences since the reactor protection system operated as designed. Last similar occurrence was reported by R.O. #84-4 on Docket #050-249.



Commonwealth Edison

Dresden Nuclear Power Station

R.R. #1

Morris, Illinois 60450

Telephone 815/242-2920

November 7, 1984

DJS Ltr #84-1261

U.S. Nuclear Regulatory Commission
Document Control Desk
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Licensee Event Report #84-017-0, Docket #050249 is being submitted as required by Technical Specification 6.6, NUREG 1022 and 10 CFR 50.73 (a)(2)(iv).

D.J. Scott
Station Superintendent
Dresden Nuclear Power Station

DJS/kjl

Enclosure

cc: J.G. Keppler, Regional Administrator, Region III
File/NRC
File/Numerical

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