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William R. Campbell
Vice President
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SERIAL: BSEP 96-0149

APR 29 1996

U. S. Nuclear Regulatory Commission
ATTENTION: Document Control Desk
Washington, DC 20555

BRUNSWICK STEAM ELECTRIC PLANT, UNIT NO. 2
DOCKET NO. 50-324/LICENSE NO. DPR-62
NRC GENERIC LETTER 94-03, "INTERGRANULAR STRESS CORROSION CRACKING OF
CORE SHROUDS IN BOILING WATER REACTORS"

Gentlemen:

On July 25, 1993, the Nuclear Regulatory Commission staff issued Generic letter (GL) 94-03, "Intergranular Stress Corrosion Cracking of Core Shrouds in Boiling Water Reactors." This generic letter requested that each licensee: (1) inspect the core shrouds in their BWR plants no later than the next scheduled refueling outage and perform an appropriate evaluation and/or repair based on the results of the inspection, and (2) perform a safety analysis supporting continued operation of the facility until inspections are conducted. Reporting Requirement 3 of Generic Letter 94-03 requires licensees to provide the results of the inspections within 30 days of the completion of the inspections.

Enclosure 1 of this letter provides the inspection results and evaluation for Carolina Power & Light Company's Brunswick Steam Electric Plant, Unit 2. These inspections were completed during the Unit 2 Refueling Outage 11 (B212R1). The Core Shroud Reinspection Plan for this refueling outage was submitted to the NRC staff by letter dated November 2, 1995 (Serial Number BSEP 95-0570).

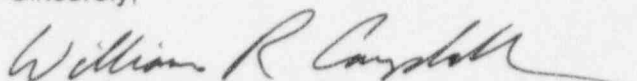
The results of the inspections completed during the Unit 2 Refueling Outage 11 indicate that the reported cracking is typical intergranular stress corrosion cracking located in the heat affected zone of the welds. Using the results obtained during these inspections, a review of the core shroud structural capacity has been performed using the criteria established by the BWRVIP in two NRC-approved documents, GE-NE-523-113-0894, "BWR Core Shroud Inspection and Flaw Evaluation Guidelines" and GE-NE-523-113-0894, Supplement 1, "BWR Core Shroud Distributed Ligament Length Computer Program." Based on the inspection results and the evaluation performed, the condition of the core shroud does not impose restrictions to the operation of Brunswick Unit 2 during this operational cycle.

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Please refer any questions regarding this submittal to Mr. George Honma at (910) 457-2741.

Sincerely,



William R. Campbell

WRM/wrm

Enclosure

cc: U. S. Nuclear Regulatory Commission
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Mr. C. A. Patterson
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U.S. Nuclear Regulatory Commission
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The Honorable H. Wells
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