

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) EDWIN I. HATCH, UNIT I	DOCKET NUMBER (2) 0 5 0 0 0 3 2 1 1	PAGE (3) 1 OF 0 5
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TITLE (4)
Failure of Components to Pass Local Leak Rate Test

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
1	0	0	8	4	0	1	1	0			0 5 0 0 0
1	0	0	8	4	0	1	1	0			0 5 0 0 0

OPERATING MODE (9) 5	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)						73.71(b)
POWER LEVEL (10) 0 0 0	20.402(b)	20.406(vi)	50.73(a)(2)(iv)			73.71(c)	
	20.406(a)(1)(i)	50.36(e)(1)	50.73(a)(2)(v)	<input checked="" type="checkbox"/>		OTHER (Specify in Abstract below and in Text, NRC Form 366A)	
	20.406(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)				
	20.406(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)				
	20.406(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)				
	20.406(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(ix)				

LICENSEE CONTACT FOR THIS LER (12)

NAME T. L. Elton, Acting Superintendent of Regulatory Compliance	TELEPHONE NUMBER 9 1 2 3 6 7 + 1 7 B 1 5 1 1
AREA CODE	

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS
X	JM	ISVGO	810	Y	X	BO	ISV	P 1305	Y
X	BO	ISVW	30	Y	X	BO	ISV	L 126B	Y

SUPPLEMENTAL REPORT EXPECTED (14)

<input checked="" type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)	<input type="checkbox"/> NO	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
			0	1	1 5 8 5

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

During performance of the "PRIMARY CONTAINMENT PERIODIC TYPE B AND C LEAKAGE TESTS" procedure (HNP-1-3952) as required by Unit 1 Tech. Specs. sections 4.7.A.2.e. (1) and 4.7.A.2.f, plant personnel determined that the following primary containment valves and a primary containment penetration were leaking in excess of the limits specified in Unit 1 Tech. Specs. section 3.7.A.2.g., and the ASME Section XI criteria specified in HNP-1-3952: (1) RHR system valves E11-F007, E11-F011B, E11-F016B, E11-F025B, E11-F028B, and E11-F097B, (2) containment atmosphere control valves T48-F309 and T48-F320, (3) main steam line drain valve B21-F016, (4) core spray pump isolation valves E21-F001A and E21-F001B, (5) RCIC vacuum breaker valve E51-F104, and (6) torus purge and inerting valves T48-F311 and T48-F324. These valves will be repaired to conform to Tech. Specs. requirements, and the requirements of ASME SECTION XI prior to unit startup.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1) EDWIN I. HATCH, UNIT I	DOCKET NUMBER (2) 0 5 0 0 0 3 2 1 8 4	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
		-	0 1 7	-	0 0	2 0 5

TEXT (If more space is required, use additional NRC Form 366A's) (17)

REQUIREMENT FOR REPORT:

This report is required by 10CFR50.73(a)(2)(ii) and 10CFR50.73(a)(2)(v)(c).

PLANT CONDITIONS AT THE TIME OF THE EVENT(S):

On 10-08-84 (first event), 10-09-84 (second event), and 10-12-84 (third event) Unit one was in cold shutdown for a refueling outage.

DETAILED DESCRIPTION OF THE EVENT(S):

On 10-08-84, after tests were performed per the "PRIMARY CONTAINMENT PERIODIC TYPE B AND C LEAKAGE TESTS" procedure (HNP-1-3952), the following valves were determined to have unacceptable leakage test results per Tech. Specs. section 4.7.A.2.g:

1. Main Steam line valves B21-F022B, B21-F028A, B21-F028C, B21-F022D, B21-F022A, and B21-F022C,
2. Residual heat removal pump suction isolation valve E11-F004A,
3. High pressure core injection turbine exhaust inboard isolation valve (E41-F021),
4. Clean radwaste pump discharge valve G11-F019,
5. Drywell penumatic system check valve P70-F020,
6. Torus penetration X-201B did not meet the leakage requirements of Tech. Specs. section 4.7.A.2.e.

On 10-09-84, after tests were performed per HNP-1-3952, the following valves were determined to have unacceptable leakage test results per Tech. Specs. section 4.7.A.2.g:

1. RHR system valves E11-F007B, E11-F011B, E11-F016B, E11-F025B, E11-F028B, and E11-F097,
2. Containment atmosphere control valve T48-F309 at penetration X-25, and containment atmosphere control valve T48-F320 at penetration X-26.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1) EDWIN I. HATCH, UNIT I	DOCKET NUMBER (2) 0 5 0 0 0 3 2 1	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		8 4	- 0 1 7	- 0 0	0 3	OF	0 5

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On 10-12-84, after tests were performed per HNP-1-3952, the following valves were determined to have unacceptable results per Tech. Specs. section 4.7.A.2.g:

1. Main steam line drain inboard containment isolation valve B21-F016,
2. Core spray pump to torus outboard suction isolation valves E21-F001A and E21-F001B,
3. RCIC vacuum breaker torus isolation valve E51-F104,
4. Torus pressure equalization valve T48-F311,
5. Torus purge and inerting system isolation valve T48-F324 at penetration X-25.

SUMMARY ASSESSMENT OF ACTUAL AND POTENTIAL SAFETY CONSEQUENCES AND IMPLICATIONS:

The consequences of this event have not been determined at this time. An engineering evaluation will be performed and an update report will be submitted.

STATUS OF REDUNDANT OR BACKUP SUBSYSTEMS AND/OR SYSTEMS:

N/A

JUSTIFICATION FOR CONTINUED OPERATION:

The valves will be repaired and satisfactorily functionally tested prior to Unit startup.

IF REPETITIVE:

NUMBER OF PREVIOUS LER:

50-321/1982-095, Rev. 1

WHY CORRECTIVE ACTION DID NOT PREVENT RECURRENCE:

Valve failures of this type are generic throughout the industry. The previous corrective action taken was sufficient at that time. However, valve failures of this type are recurrent.

IMPACT TO OTHER SYSTEMS AND/OR UNIT:

This event could have affected other systems on Unit 1. This event had no impact upon Unit 2.

CAUSE(S) OF THE EVENTS(S):

The cause of these events has not been determined at this time. An update report will be submitted within 30 days after unit startup.

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FACILITY NAME (1) EDWIN I. HATCH, UNIT I	DOCKET NUMBER (2) 0 5 0 0 0 3 2 1	LER NUMBER (6)			PAGE (3)	
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		8 4	- 0 1 7	- 0 0	0 4	OF 0 5

TEXT (If more space is required, use additional NRC Form 386A's) (17)

IMMEDIATE CORRECTIVE ACTION(S):

N/A

SUPPLEMENTAL CORRECTIVE ACTION(S):

N/A

SCHEDULED (FUTURE) CORRECTIVE ACTION(S):

The valves will be repaired prior to unit startup. The corrective action taken will be provided in an update report and submitted within 30 days after unit startup.

ACTION(S) TO PREVENT RECURRENCE (IF DIFFERENT FROM CORRECTIVE ACTIONS):

N/A

LICENSEE EVENT REPORT (LER) FAILURE CONTINUATION

FACILITY NAME (1)

DOCKET NUMBER (2)

LER NUMBER (6)

PAGE (3)

EDWIN I. HATCH, UNIT I

0 | 5 | 0 | 0 | 0 | 3 | 2 | 1 | 8 | 4 | - | 0 | 1 | 7 | - | 0 | 0 | 0 | 5 | OF | 0 | 5

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFAC-TURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFAC-TURER	REPORTABLE TO NPRDS
X	B D	I S V	G 0 8 0	Y					
X	B G	I S V	A 5 8 5	Y					
X	B N	I S V	V 0 8 5	Y					
X	B P	I S V	F 1 3 0	Y					

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912 537-9444



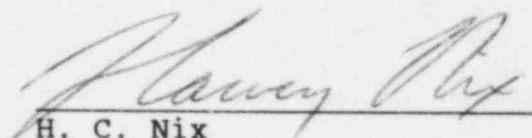
Edwin I. Hatch Nuclear Plant

November 1, 1984
GM-84-985

PLANT E. I. HATCH
Licensee Event Report
Docket No. 50-321

United States Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555

Attached is Licensee Event Report No. 50-321/1984-017. This report is required by 10CFR 50.73(a)(2)(v).


H. C. Nix
General Manager

HCN/TLE/vlz

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1/1