

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) <b>EDWIN I. HATCH, UNIT II</b>	DOCKET NUMBER (2) 0   5   0   0   0   3   6   6	PAGE (3) 1 OF 02
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TITLE (4)  
**Reactor Scram Due To Erratic Instruments**

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
1	0	0	8	4	9	4	0	2	6	0	0
1	0	0	4	8	4	9	4	0	2	6	0
0	0	1	1	0	2	8	4				

OPERATING MODE (9) 1	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)									
POWER LEVEL (10) 0   9   9	<input type="checkbox"/> 70.40(a)	<input type="checkbox"/> 20.406(c)	<input checked="" type="checkbox"/> 50.73(a)(2)(iv)	<input type="checkbox"/> 73.71(b)						
	<input type="checkbox"/> 20.406(a)(1)(i)	<input type="checkbox"/> 50.36(c)(1)	<input type="checkbox"/> 50.73(a)(2)(v)	<input type="checkbox"/> 73.71(c)						
	<input type="checkbox"/> 20.406(a)(1)(ii)	<input type="checkbox"/> 50.36(c)(2)	<input type="checkbox"/> 50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)						
	<input type="checkbox"/> 20.406(a)(1)(iii)	<input type="checkbox"/> 50.73(a)(2)(i)	<input type="checkbox"/> 50.73(a)(2)(viii)(A)							
	<input type="checkbox"/> 20.406(a)(1)(iv)	<input type="checkbox"/> 50.73(a)(2)(ii)	<input type="checkbox"/> 50.73(a)(2)(viii)(B)							
	<input type="checkbox"/> 20.406(a)(1)(v)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(x)							

LICENSEE CONTACT FOR THIS LER (12)		TELEPHONE NUMBER	
NAME <b>T. L. Elton, Acting Superintendent of Regulatory Compliance</b>		AREA CODE 9   1   2	NUMBER 3   6   7   1   7   8   5   1

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)	<input checked="" type="checkbox"/> NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 10/04/84 at approximately 1902 CST, the reactor scrambled due to a turbine trip above 30% power. The turbine trip resulted from a moisture separator reheater (MSR) high level trip instrumentation actuation. On 10/07/84 at approximately 2000 CST, the reactor again scrambled from a turbine trip above 30% power due to MSR high level instrumentation actuation.

The cause of the MSR high level trip was apparently due to the MSR reference leg level on the MSR level transmitters (2N21-N304 A-D and 2N21-N305 A-D) varying in level instead of remaining filled.

A Design Change Request (DCR) was implemented to resolve the reference leg level problem. This DCR changed the purge on the reference leg to ensure the reference leg remains filled at any tank level or turbine load.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1)  EDWIN I. HATCH, UNIT II	DOCKET NUMBER (2)  0 5 0 0 0 3 6 6 8 4	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
			- 0 2 6	- 0 0 0	2	OF	2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

This 30-day report is required by 10CFR50.73 (a)(2)(iv) due to these events resulting in the unplanned actuation of an ESF (RPS).

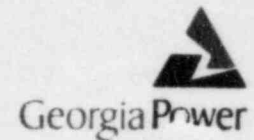
On 10/04/84 at approximately 1902 CST, the unit was in steady-state operation at approximately 2409 Mwt (approximately 99% power) and on 10/07/84 at approximately 2000 CST the unit was in steady-state operation at approximately 2204 Mwt (approximately 90% power).

On 10/04/84 at approximately 1902 CST, the reactor scrammed due to a turbine trip above 30% power. The turbine trip resulted from a moisture separator reheater (MSR) high level trip instrumentation actuation. On 10/07/84 at approximately 2000 CST, the reactor again scrammed from a turbine trip above 30% power due to MSR high level instrumentation actuation.

The cause of the MSR high level trip was apparently due to the MSR reference leg level on the MSR level transmitters (2N21-N304 A-D and 2N21-N305 A-D) varying in level instead of remaining filled.

A Design Change Request (DCR) was implemented to resolve the reference leg level problem. This DCR changed the purge on the reference leg to ensure the reference leg remains filled at any tank level or turbine load.

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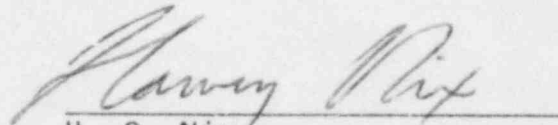
Edwin I. Hatch Nuclear Plant

November 2, 1984  
GM-84-970

PLANT E. I. HATCH  
Licensee Event Report  
Docket No. 50-366

United States Nuclear Regulatory Commission  
Document Control Desk  
Washington, D. C. 20555

Attached is Licensee Event Report No. 50-366/1984-026. This report is required by 10CFR 50.73 a)(2)(iv).

  
H. C. Nix  
General Manager

HCN/TLE/vlz

xc: R. J. Kelly  
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1/1