50-302



## UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D.C. 20555-0001

May 2, 1996

Mr. Percy M. Beard, Jr. Senior Vice President, Nuclear Operations (SA2A) Florida Power Corporation ATTN: Manager, Nuclear Licensing 15760 W Power Line Street Crystal River, Florida 34428-6708

SUBJECT: CRYSTAL RIVER NUCLEAR GENERATING PLANT UNIT 3 - EVALUATION OF FLORIDA POWER CORPORATION'S PLANT-SPECIFIC CRITERIA AND PROCEDURES FOR IMPLEMENTING THE RESOLUTION OF USI A-46 (GENERIC LETTER &7-02) AT CRYSTAL RIVER UNIT 3 (TAC NO. M69440)

Dear Mr. Beard:

The staff has completed its evaluation of your submittal relating to plantspecific criteria and procedures for verifying the seismic adequacy of equipment at Crystal River Unit 3 (CR-3) for the resolution of unresolved safety issue (USI A-46) as delineated in Generic Letter (GL) 87-02. Our evaluation is based primarily on the information contained in your submittals dated August 27, 1993 as supplemented August 15, and September 16, 1994, and additional supporting and clarifying information provided by your staff during discussions with us during our review process.

Our Safety Evaluation (SE) is enclosed. Please note that our SE discusses only the acceptability of the plant-specific criteria and procedures used by the licensee, and does not include your plant-specific walkdown. Based on our review, we find that your program is generally adequate to resolve the primary concern of USI A-46. We also plan to determine whether your USI A-46 program conforms to the intent of GL 87-02 when we complete our review of your final summary report documenting the results of your plant-specific walkdown.

Please note that our SE is subject to you confirming that the equipment necessary to assure core decay heat removal for 72 hours in both of the safe shutdown paths are seismically adequate, and developing a top-level procedure for coping with the consequences of relay chatter. Additonally, section 3.0 of our SE discusses several open issues in your approach for evaluating the seismic adequacy of equipment anchorage, cable and conduit raceways, tanks, and the generic caveats for equipment classes. These open issues will be a subject of our future inspection and audits. These issues were previously discussed with your staff.

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Percy M. Beard, Jr.

Please note that our review of your plant-walkdown results is in progress. Results of this review will be documented in a supplemental SE.

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Sincerely,

L. Raghavan, Project Manager Project Directorate II-3 Division of Reactor Projects - I/II Office of Nuclear Reactor Regulation

Docket No. 50-302

Enclosure: Safety Evaluation

cc w/enclosure: See next page

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