

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) **Susquehanna Steam Electric Station - Unit 1** DOCKET NUMBER (2) **05000387** PAGE (3) **1 OF 02**

TITLE (4) **Auxiliary Boiler Arc-Over/Primary Containment Isolation Valves Closed.**

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)	
10	06	84	84	043	00	11	05	84		05000	
										05000	

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 50.73 (Check one or more of the following) (11)

OPERATING MODE (9)	1	20.402(b)	20.408(a)	<input checked="" type="checkbox"/>	80.73(a)(2)(w)	73.71(b)
POWER LEVEL (10)	0.61	20.408(a)(1)(i)	50.38(e)(1)	<input type="checkbox"/>	80.73(a)(2)(v)	73.71(e)
		20.408(a)(1)(ii)	50.38(e)(2)	<input type="checkbox"/>	80.73(a)(2)(u)	OTHER (Specify in Abstract below and in Text, NRC Form 365A)
		20.408(a)(1)(iii)	80.73(a)(2)(i)	<input type="checkbox"/>	80.73(a)(2)(viii)(w)	
		20.408(a)(1)(iv)	80.73(a)(2)(ii)	<input type="checkbox"/>	80.73(a)(2)(viii)(u)	
		20.408(a)(1)(v)	80.73(a)(2)(iii)	<input type="checkbox"/>	80.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME **L.A. Kuczynski - Nuclear Plant Specialist, Level III** TELEPHONE NUMBER **7117 51421-1371519**

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC
B	SA	BLR	H3218						

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE)  NO

EXPECTED SUBMISSION DATE (15) MONTH **02** DAY **01** YEAR **85**

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

The Station's auxiliary steam is supplied by two (2) electrode boilers. On October 6, 1984, auxiliary boiler 'B' experienced an internal arc-over. The resultant overcurrent tripped the 13.8KV breaker which supplies the boiler. This caused a transient on Startup Bus 20 which resulted in a variety of alarms and system perturbations, including the closure of valves associated with the Unit 1 Containment Radiation Monitoring system 'B' and the Reactor Water Cleanup Containment Outboard Isolation valve. These valves are part of the Primary Containment Isolation System, which is an Engineered Safety Feature. All affected equipment was returned to normal. Long-term actions to prevent recurrence are still being formulated and will be reported as an update to this LER.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1) Susquehanna Steam Electric Station Unit 1	DOCKET NUMBER (2) 05000387	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
		84	043	00	02	OF 02

TEXT (If more space is required, use additional NRC Form 366A's) (17)

The Station's auxiliary steam is supplied by two electrode boilers. On October 6, 1984, auxiliary boiler 'B' experienced an internal arc-over, which resulted in the trip of the 13.8KV breaker which supplies the boiler. This caused a transient on Startup Bus 20 which resulted in the isolation of the Unit 1 containment radiation monitoring system 'B', miscellaneous alarms, loss of power to the Main Steam Isolation Valve-Leakage Control System (Inboard) system, lock-up of the Unit 1 Reactor Recirculation Pump Motor-Generator Set scoop tubes and the closure of the Reactor Water Clean-up (RWCU) System Outboard Isolation valve. The RWCU valve and the valves which are associated with the Containment Radiation Monitoring system 'B' are part of the Primary Containment Isolation System, which is an Engineered Safety Feature. The Station was returned to normal without incident.

Auxiliary boiler 'B' was drained and thoroughly inspected. The results of the inspection were indeterminate. A similar occurrence was reported in Licensee Event Report (LER) 82-065/03L-0. Commitments to install isolation transformers and more accurate conductivity meters were made in that LER. PP&L has elected to forego the installation of the isolation transformers because they are not an effective solution. In response to the October 6, 1984 event, eroded counter electrodes have been removed and reworked, outer insulators are being replaced and inner insulators cleaned. Boiler conductivity both before and after the event was within specifications. Long-term corrective actions to prevent recurrence are still being formulated and will be reported as an update to this LER.



SUSQUEHANNA STEAM ELECTRIC STATION  
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November 5, 1984

U.S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, DC 20555

SUSQUEHANNA STEAM ELECTRIC STATION  
LICENSEE EVENT REPORT 84-043-00  
ER 100450 FILE 841-23  
PLAS-006

Docket No. 50-387  
License No. NPF-14

Attached is Licensee Event Report 84-043-00. This event was determined reportable per 10CFR50.73(a)(2)(iv), in that the unit experienced an unanticipated Engineered Safety Feature actuation when various Primary Containment Isolation System valves closed.

H.W. Keiser  
Superintendent of Plant-Susquehanna

LAK/pjg

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