

Tennessee Valley Authority. Post Office Box 2000, Decatur, Alabuma 35609-2000

R. D. (Rick) Machon Vice Prosident, Browns Ferry Nuclear Plant

April 26, 1996

U.S. Nuclear Regulatory Commission ATTN: Document Control Desk Washington, D.C. 20:35

10 CFR 50.73

Dear Sir:

BROWNS FLOAT NUCLEAR PLANT (BFN) - UNITS 1, 2, AND 3 - DOCKET NOS. 50-259, 260, and 296 - FACILITY OPERATING LICHNSE DPR-33, 52, AND 68 - LICENSEE EVENT REPORT 50-259/06002

The enclosed report provides details concerning an unplanned emergency diesel generator start. The emergency diesel generator auto-started on undervoltage condition following an unexpected trip of a shutdown board supply breaker during testing. This report is submitted in accordance with 10 CFR 50.73(a)(2)(iv) as a condition that resulted in manual or automatic actuation of an engineered safety feature.

Sincerely,

R. D. Maghon

cc: See page 2

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Enclosure cc (Enclosure): Mr. Mark S. Lesser, Branch Chief U.S. Nuclear Regulatory Commission Region II 101 Marietta Street, NW, Suite 2900 Atlanta, Georgia 30323

> NRC Resident Inspector Browns Ferry Nuclear Plant 10833 Shaw Road Athens, Alabama 35611

Mr. J. F. Williams, Project Manager U.S. Nuclear Regulatory Commission One White Flint, North 11555 Rockville Pike Rockville, Maryland 20852

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board D deenergized, EDG D auto-started and fied onto the board. The deenergized board also resulted in a reactor protection system (RPS) half-scram, a loss of power to RPS bus 2B, and isolation of primary containment isolation system group 6 (ventilation related). The root cause was personnel error. TVA's human performance evaluation of the technician revealed habit intrusion that resulted in failure to follow instruction. Specifically, the technician did not follow the testing instruction as a result of mindset created from similar tests performed prior to this event. Immediate corrective actions included restoring power to the affected board and securing the EDG. The technician involved was bounselled. Additionally, as an enhancement, a caution note regarding the potential for engineered safety feature actuation associated with 52STA switch operation will be added to the appropriate testing instructions. There was a previous LER (259/ 93002) that also resulted in an EDG start from breaker cell switch actuation; however, corrective actions taken in LER 259/93002 would not have precluded this event. NRC FORM 366A

(4-95)

#### U.S. NUCLEAR REGULATORY COMMISSION

# LICENSEE EVENT REPORT (LER)

TEXT CONTINUATION

FACILITY NAME (1)	DOCKET		LER NUMBER	(6)	PAGE (3)
n		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
Browns Ferry Unit 1	05000259	96 -	96 002 00	2 of 5	

TEXT (If more space is required, use additional apples of NRC Form 366A) (17)

### I. PLANT CONDITIONS

At the time of this event, Unit 1 was shutdown and defueled. Unit 2 was in cold shutdown for a scheduled refueling outage. Unit 3 was operating at 100 percent power.

## II. DESCRIPTION OF EVENT

### A. Event:

On March 28, 1996, at 1205 hours, during functional tests on breakers associated with 4kV shutdown (SD) [EB] bus 1, Unit 1/2 emergency diesel generator (EDG) [EK] D auto-started on undervoltage condition following an unexpected trip of the 4kV SD board D normal supply breaker [BKR].

At the time of this event, a functional check was being performed in the breaker compartment for the 4kV SD board D alternate supply breaker. After racking the alternate supply breaker to DISCONNECTED position and blocking the breaker's position switch (33), a technician [utility, nonlicensed] inadvertently activated the breaker's stationary auxiliary (cell) switch (52STA). The activated switch, which interlocks the normal and alternate supply breakers to prevent paralleling power sources, resulted in a trip of the normal supply breaker, as designed. With the alternate supply breaker disconnected, the trip of the normal supply breaker deenergized 4kV SD board D. Unit 1/2 EDG D autostarted on undervoltage and tied onto the board.

The deenergized 4kV SD board D resulted in a reactor protection system (RPS) [JC] half-scram, a loss of power to RPS bus 2B, and actuation or isolation of primary containment isolation system (PCIS) [JE] logic channels powered from RPS 2B (i.e., Group 6 -Primary Containment Purge and Ventilation [JM], Reactor Zone Ventilation [VB], Refueling Zone [VA], Standby Gas Treatment (BH], and Control Room Emergency Ventilation [VI]). (Other PCIS groups were already isolated or bypassed for outage work.)

At 1222 hours, RPS 2B power was restored, and the half-scram and PCIS isolations were reset. At approximately 1237 hours, Unit 1/2 EDG D was shutdown and returned to standby readiness.

This event is reportable pursuant to 10 CFR 50.73(a)(2)(1v) as any event or condition that resulted in manual or automatic actuation of an engineered safety feature (ESF) [JE], including the RPS.

# B. Inoperable Structures, Components, or Systems that Contributed to the Event:

None.

NRC FORM 366A U.S. NUCLEAR REGULATORY COMMISSION (4-95) LICENSEE EVENT REPORT (LER) TEXT CONTINUATION FACILITY NAME (1) DOCKET LER NUMBER (6) YEAR SEQUENTIAL REVISION NUMBER NUMBER Browns Ferry Unit 1 05000259 96 002 00 TEXT (If more space is required, use additional copies of NRC Form 366A) (17) C. Dates and Approximate Times of Major Occurrences: March 28, 1996

at 1205 hours	Unit 1/2 EDG D auto-started. RPS half-scram and PCIS group 6 received.
at 1222 hours	RPS half-scram and PCIS isolations reset.
at 1237 hours	Unit 1/2 EDG D shutdown and returned to standby readiness.
at 1431 hours	TVA made a 4-hour nonemergency notification to NRC in accordance with 10 CFR 50.72(b)(2)(ii).

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#### D. Other Systems or Secondary Functions Affected:

None.

#### Method of Discovery: Ε.

This condition was discovered when the control room Operations personnel [utility, licensed] received alarms and indicators that 4kV SD board D was deenergized and Unit 1/2 EDG D had autostarted.

#### F. Operator Actions:

Operations personnel responded to the event in accordance with approved procedures. The half-scram and PCIS isolations were reset. Normal power was returned to the board and the EDG secured.

#### G. Safety System Responses:

The plant systems responded as designed for loss of power to a shutdown board.

#### III. CAUSE OF THE EVENT

#### Immedizte Cause: A .

The immediate cause was the actuation of the alternate supply breaker's 52STA switch. The actuation of the switch in the alternate supply breaker compartment gave an indication to the normal supply breaker trip logic that the alternate supply breaker had closed. As designed, the normal supply breaker opened.

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# LICENSEE EVENT REPORT (LER)

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TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

## B. Root Cause:

The root cause of this event was personnel error. Human performance evaluation of the technician revealed habit intrusion that resulted in failure to follow instruction. Specifically, the technician did not follow the Specific Maintenance Instruction (SMI) in testing the 4kV supply breaker as a result of mindset (habit intrusion) created from performing similar tests prior to this event.

The technician had recently participated (within the last 5 days) in functional testing of 4kV SD boards A and B. Since each of these tests required breaker test linkages to be installed in the breaker compartments, the technician became habituated to the use of test linkages for testing breakers. Therefore, a mindset was created in that test linkages are needed for the 4kV SD board supply breaker tests. Contrary to the steps in the SMI, a test linkage was installed in the alternate supply breaker compartment. During the placement of this linkage, the technician actuated the 52STA switch.

# IV. ANALYSIS OF THE EVENT

As expected, the auto-start of the EDG was the successful completion of a designed function. The normal and alternate supply breakers for the shutdown board are interlocked using the 52STA switch to prevent paralleling power sources. An EDG is primarily assigned to each 4kV shutdown board. When the shutdown board is deenergized, the associated EDG breaker closes tying the EDG to its shutdown board. In this event, all expected automatic actions took place, and plant safety systems and associated components performed as designed.

At the time of this event, Unit 2 was in a refueling outage. The shutdown bus functional check was immediately stopped, and the cause of the power loss was identified. Normal power was returned to the board without losing any shutdown cooling. Operations personnel took appropriate corrective actions to restore the plant systems to standby readiness. Therefore, the safety of the plant, its personnel, and the public was not compromised.

### V. CORRECTIVE ACTIONS

### A. Immediate Corrective Actions:

Functional tests on breakers associated with the shutdown bus were stopped. Operations personnel restored power to 4kV SD board D. The EDG was shutdown and placed in standby readiness. A problem evaluation report (BFPER960326) was generated to document this event. NRC FORM 366A

U.S. NUCLEAR REGULATORY COMMISSION

# LICENSEE EVENT REPORT (LER)

TEXT CONTINUATION

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# B. Corrective Actions to Prevent Recurrence:

The technician was counselled to further emphasize the need to follow instruction and to implement the five steps of 'STA<sup>2</sup>R' (i.e., Stop, Think, Ask, Act, and Review) work practices into switching and testing procedures.

As an enhancement, a caution note regarding the potential for ESF actuation associated with 52STA switch operation will be added to the appropriate SMIs.<sup>1</sup> The note will be added to the SMIs prior to testing of the interlocking breakers.

### VI. ADDITIONAL INFORMATION

### A. Failed Components:

None.

# B. Previous LERs on Similar Events:

One previous LER (259/93002) resulted from actuation of a breaker cell switch. The personnel involved did not realize that breaker compartment maintenance would operate the cell switch in the breaker compartment. The corrective actions taken in LER 259/93002 involved personnel training and addition of a caution note concerning ESF actuation in the electrical preventive maintenance instructions. However, these corrective actions would not have precluded this event (LER 259/96002) since the technician involved in installing the test linkage knew 52STA switch manipulation can actuate the switch, and the training conducted in LER 259/93002 did not include the technician's group nor was the caution note added to the SMIs.

#### VII. COMMITMENTS

None.

Energy Industry Identification System (EIIS) system and component codes are identified in the text with brackets (e.g., [XX]).

<sup>1</sup> This is not a regulatory commitment.

NRC FORM 365A (4-95)