NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
EDWIN I. HATCH, UNIT 2	0 5 0 0 0 3 6 6	1 OF 0 2

TITLE (4)

ESF ACTUATION DUE TO HIGH DIFFERENTIAL FLOW

EVENT DATE (5)			LER NUMBER (6)					REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR		SEQUENTIAL NUMBER		REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
												0 5 0 0 0
0 9	2 4	8 4	8 4	—	0 1 0	—	0 1	1 1	0 5	8 4		0 5 0 0 0

OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)			
1		20.402(b)	20.405(c)	X 50.73(a)(2)(iv)	73.71(b)
POWER LEVEL (10)		20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
0 7 2		20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
		20.405(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
		20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
		20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
	AREA CODE
T. L. Elton, Acting Supt. of Regulatory Compliance	9 1 2 3 6 7 - 7 8 5 1

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 09/08/84, at approximately 1500 CDT, with the reactor mode switch in the run position and reactor power at 1745 MWT (approximately 72% power), the Reactor Water Clean-up (RWCU) Inboard and Outboard Isolation Valves (2G31-F001 and 2G31-F004, respectively) isolated due to a high differential flow signal. No actual or potential safety consequences or implications resulted from this event. This event had no impact on any other Unit 2 systems or on Unit 1. The health and safety of the public were not affected by this non-repetitive event.

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1/1

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
EDWIN I. HATCH, UNIT 2	0 5 0 0 0 3 6 6	8 4	— 0 1 0	— 0 1	0 2 OF 0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

This 30 day LER is required by 10CFR 50.73 (a)(2)(IV) since the Reactor Water Clean-up Inboard and Outboard Isolation Valves (2G31-F001 and 2G31-F004, respectively) are Primary Containment Isolation Valves and their isolation constitutes the actuation of an Engineered Safety Feature (ESF).

On 09/08/84, at approximately 1500 CDT, with the reactor mode switch in the run position and reactor power at 1745 MWT (approximately 72% power), the Reactor Water Clean-up (RWCU) Inboard and Outboard Isolation Valves (2G31-F001 and 2G31-F004, respectively) isolated due to a high differential flow signal. No actual or potential safety consequences or implications resulted from this event. This event had no impact on any other Unit 2 systems or on Unit 1. The health and safety of the public were not affected by this non-repetitive event.

An investigation concluded that when the "A" RWCU demineralizer was manually valved in (after a backwash and precoat), the demineralizer was not completely full. Thus, the high differential flow signal and subsequent isolation signal were received. The RWCU System was returned to service at approximately 1520 CDT on 09/08/84.

On 09/24/84, while performing an investigation to write another LER, plant personnel determined that the above event had occurred. On 09/26/84 the determination was made that a Deficiency Report had not been written due to personnel oversight. A Deficiency Report was filed and the 4 hour notification as required by 10CFR 50.72 (b)(2)(ii) was made. Thus, the discovery date for this event was 09/24/84.

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912 537-9444

Edwin I. Hatch Nuclear Plant

Georgia Power

November 5, 1984
GM-84-994

PLANT E. I. HATCH
Licensee Event Report
Docket No. 50-366

United States Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Attached is Licensee Event Report No. 50-366/1984-10, Rev. 1. This report is required by 10CFR 50.73(a)(2)(IV).

H. C. Nix
General Manager

HCN/TLE/vlt
TKS

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