

OPERATING DATA REPORT

DOCKET NO. 50-315
 DATE 10/2/84
 COMPLETED BY Cliner
 TELEPHONE 616-465-5901

OPERATING STATUS

1. Unit Name: Donald C. Cook 1
2. Reporting Period: September 1984
3. Licensed Thermal Power (MWe): 3250
4. Nameplate Rating (Gross MWe): 1152
5. Design Electrical Rating (Net MWe): 1030
6. Maximum Dependable Capacity (Gross MWe): 1056
7. Maximum Dependable Capacity (Net MWe): 1020
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

Notes

9. Power Level To Which Restricted, If Any (Net MWe): _____
10. Reasons For Restrictions, If Any: _____

	This Month	Yr.-to-Date	Cumulative
11. Hours in Reporting Period	720	6,575	85,463
12. Number Of Hours Reactor Was Critical	720	5,866.8	63,484.8
13. Reactor Reserve Shutdown Hours	0	0	463
14. Hours Generator On-Line	720	5,808.8	62,152.5
15. Unit Reserve Shutdown Hours	0	0	321
16. Gross Thermal Energy Generated (MWH)	2,229,693	17,754,597	183,448,310
17. Gross Electrical Energy Generated (MWH)	721,400	5,792,250	59,718,540
18. Net Electrical Energy Generated (MWH)	694,907	5,577,661	57,458,037
19. Unit Service Factor	100	88.3	74.5
20. Unit Availability Factor	100	88.3	74.5
21. Unit Capacity Factor (Using MDC Net)	94.6	83.2	67.6
22. Unit Capacity Factor (Using DER Net)	93.7	82.4	64.9
23. Unit Forced Outage Rate	0	6.7	7.7

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
Refueling and 10 year outage tentatively scheduled for March 19, 1985, 120 days.

25. If Shut Down At End Of Report Period, Estimated Date of Startup: _____

	Forecast	Achieved
INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICITY	_____	_____
COMMERCIAL OPERATION	_____	_____

B411160002 840930
 PDR ADDCK 05000315
 R PDR

IE24
 11/14/84

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-315

UNIT 1

DATE 10/2/84

COMPLETED BY D. Climer

TELEPHONE (616)465-5901

MONTH September 1984

DAY	AVERAGE DAILY POWER LEVEL (MWE-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	495	17	1018
2	508	18	1018
3	635	19	1018
4	937	20	1016
5	1010	21	1015
6	1011	22	1016
7	1008	23	1015
8	1012	24	1014
9	1011	25	1013
10	1012	26	1016
11	1011	27	1018
12	1011	28	1019
13	1010	29	1018
14	1010	30	1018
15	1019	31	-
16	1022		

INSTRUCTIONS

On this format list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole number.

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-315
 UNIT NAME D.C. Cook - Unit 1
 DATE October 4, 1984
 COMPLETED BY B.A. Svensson
 TELEPHONE 616/465-5901

REPORT MONTH September, 1984

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
229 Cont'd	840831	F	0	B	4	N.A.	HH	TURBIN	<p>The power reduction which started on 840831 to repair the east main feed pump turbine inboard bearing continued until 840902 when the east MFPT was returned to service. 100% reactor power was reached on 840904.</p> <p>There were no other Unit shutdowns or significant power reductions during the month.</p>

¹
 F: Forced
 S: Scheduled

²
 Reason:
 A Equipment Failure (Explain)
 B Maintenance or Test
 C Refueling
 D Regulatory Restriction
 E Operator Training & License Examination
 F Administrative
 G Operational Error (Explain)
 H Other (Explain)

³
 Method:
 1 Manual
 2 Manual Scram.
 3 Automatic Scram.
 4 Other (Explain)

⁴
 Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

⁵
 Exhibit I - Same Source

UNIT SHUTDOWNS AND POWER REDUCTIONS

INSTRUCTIONS

This report should describe all plant shutdowns during the report period. In addition, it should be the source of explanation of significant dips in average power levels. Each significant reduction in power level (greater than 20% reduction in average daily power level for the preceding 24 hours) should be noted, even though the unit may not have been shut down completely¹. For such reductions in power level, the duration should be listed as zero, the method of reduction should be listed as 4 (Other), and the Cause and Corrective Action to Prevent Recurrence column should explain. The Cause and Corrective Action to Prevent Recurrence column should be used to provide any needed explanation to fully describe the circumstances of the outage or power reduction.

NUMBER. This column should indicate the sequential number assigned to each shutdown or significant reduction in power for that calendar year. When a shutdown or significant power reduction begins in one report period and ends in another, an entry should be made for both report periods to be sure all shutdowns or significant power reductions are reported. Until a unit has achieved its first power generation, no number should be assigned to each entry.

DATE. This column should indicate the date of the start of each shutdown or significant power reduction. Report as year, month, and day. August 14, 1977 would be reported as 770814. When a shutdown or significant power reduction begins in one report period and ends in another, an entry should be made for both report periods to be sure all shutdowns or significant power reductions are reported.

TYPE. Use "F" or "S" to indicate either "Forced" or "Scheduled," respectively, for each shutdown or significant power reduction. Forced shutdowns include those required to be initiated by no later than the weekend following discovery of an off-normal condition. It is recognized that some judgment is required in categorizing shutdowns in this way. In general, a forced shutdown is one that would not have been completed in the absence of the condition for which corrective action was taken.

DURATION. Self-explanatory. When a shutdown extends beyond the end of a report period, count only the time to the end of the report period and pick up the ensuing down time in the following report periods. Report duration of outages rounded to the nearest tenth of an hour to facilitate summation. The sum of the total outage hours plus the hours the generator was on line should equal the gross hours in the reporting period.

REASON. Categorize by letter designation in accordance with the table appearing on the report form. If category H must be used, supply brief comments.

METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER. Categorize by number designation

¹Note that this differs from the Edison Electric Institute (EEI) definitions of "Forced Partial Outage" and "Scheduled Partial Outage." For these terms, EEI uses a change of 30 MW as the break point. For large power reactors, 30 MW is too small a change to warrant explanation.

in accordance with the table appearing on the report form. If category 4 must be used, supply brief comments.

LICENSEE EVENT REPORT #. Reference the applicable reportable occurrence pertaining to the outage or power reduction. Enter the first four parts (event year, sequential report number, occurrence code and report type) of the five part designation as described in Item 17 of Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161). This information may not be immediately evident for all such shutdowns, of course, since further investigation may be required to ascertain whether or not a reportable occurrence was involved.) If the outage or power reduction will not result in a reportable occurrence, the positive indication of this lack of correlation should be noted as not applicable (N/A).

SYSTEM CODE. The system in which the outage or power reduction originated should be noted by the two digit code of Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161).

Systems that do not fit any existing code should be designated XX. The code ZZ should be used for those events where a system is not applicable.

COMPONENT CODE. Select the most appropriate component from Exhibit I - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161), using the following criteria:

- A. If a component failed, use the component directly involved.
- B. If not a component failure, use the related component; e.g., wrong valve operated through error; list valve as component.
- C. If a chain of failures occurs, the first component to malfunction should be listed. The sequence of events, including the other components which fail, should be described under the Cause and Corrective Action to Prevent Recurrence column.

Components that do not fit any existing code should be designated XXXXXX. The code ZZZZZZ should be used for events where a component designation is not applicable.

CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE. Use the column in a narrative fashion to amplify or explain the circumstances of the shutdown or power reduction. The column should include the specific cause for each shutdown or significant power reduction and the immediate and contemplated long term corrective action taken, if appropriate. This column should also be used for a description of the major safety-related corrective maintenance performed during the outage or power reduction including an identification of the critical path activity and a report of any single release of radioactivity or single radiation exposure specifically associated with the outage which accounts for more than 10 percent of the allowable annual values.

For long textual reports continue narrative on separate paper and reference the shutdown or power reduction for this narrative.

Docket No.: 50-315
Unit Name: D.C. Cook Unit 1
Completed By: G. J. Peak
Telephone: (616) 465-5901
Date: 10/04/84
Page: 1 of 2

MONTHLY OPERATING ACTIVITIES - SEPTEMBER, 1984

HIGHLIGHTS:

The unit entered the reporting period in Mode 1 with the reactor at 57% power due to the East Main Feed Pump being out of service. The East Main Feed Pump was returned to service and the unit was subsequently loaded to 100% power where it was operating when the reporting period came to an end. No other major power reductions occurred during the reporting period.

Total electrical generation for the month was 721,400 MWH.

SUMMARY:

- 9-02-84 The East Main Feed Pump was returned to service at 2053 hours and a power increase to 100% began.
- 9-04-84 The unit reached 100% power at 0940 hours.
- 9-05-84 Engineering safety features ventilation unit HV-AES-2 was inoperable from 0618 hours on 9-05-84 to 1446 hours on 9-10-84 for repair of the dampers and door frame.
- 9-20-84 The Spray Additive Tank was inoperable for a 12 hour period for maintenance.

The Control Room Cable Vault Halon System remains inoperable as of 1400 hours on 4-05-83. The backup CO₂ System for the Control Room Cable Vault remains operable.

DOCKET NO.	<u>50 - 315</u>
UNIT NAME	<u>D. C. Cook - Unit No. 1</u>
DATE	<u>10-5-84</u>
COMPLETED BY	<u>B. A. Svensson</u>
TELEPHONE	<u>(616) 465-5901</u>
PAGE	<u>1 of 1</u>

MAJOR SAFETY-RELATED MAINTENANCE

SEPTEMBER, 1984

- M-1 IMG-51, BIT Discharge to Loop #1 600V ground was found to be caused by a pinched motor lead in the limiter operator switch compartment.
- M-2 U-1 CD diesel starting air compressor discharge check valve #1-CD-1 was leaking by. The valve was disassembled and the seat lapped and blued to restore seating.
- C&I-1 Bistable LB-519A/B in the steam generator mismatch protection circuit was replaced due to repeated setpoint drift. It was recalibrated and the surveillance test was performed to verify operability.



INDIANA & MICHIGAN ELECTRIC COMPANY

Donald C. Cook Nuclear Plant
P.O. Box 458, Bridgman, Michigan 49106

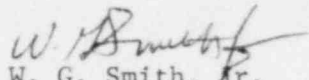
October 5, 1984

Director, Office Of Management Information
and Program Control
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Gentlemen:

Pursuant to the requirements of Donald C. Cook Nuclear Plant Unit 1
Technical Specification 6.9.1.6, the attached Monthly Operating
Report for the Month of September, 1984 is submitted.

Sincerely,


W. G. Smith, Jr.
Plant Manager

WGS:ab

Attachments

cc: J. E. Dolan
M. P. Alexich
R. W. Jurgensen
NRC Region III
E. R. Swanson
R. O. Bruggee (NSAC)
R. C. Callen
S. J. Mierzwa
R. F. Kroeger
B. H. Bennett
J. D. Huebner
J. H. Hennigan
Z. Cordero
R. F. Hering
J. F. Stietzel
PNSRC File
INPO Records Center
ANI Nuclear Engineering Department

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