

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-346

UNIT Davis-Besse #1

DATE 10-2-84

COMPLETED BY Bilal M. Sarsour

TELEPHONE (419) 259-5000
Ext. 384

MONTH September, 1984

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	619
2	618
3	619
4	619
5	619
6	619
7	619
8	591
9	565
10	564
11	276
12	0
13	0
14	0
15	0
16	0

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
17	0
18	0
19	0
20	0
21	0
22	0
23	0
24	0
25	0
26	0
27	0
28	0
29	0
30	0
31	-

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

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UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH September, 1984

DOCKET NO. 50-346
 UNIT NAME Davis-Besse #1
 DATE 10-2-84
 COMPLETED BY Bilal M. Sarsour
 TELEPHONE (419)259-5000 Ext. 384

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
5	84-09-11	F	59.4	G	3	NP-33-84-13	CD	Instru	The reactor tripped by the Anticipatory Reactor Trip System (ARTS) in response to the turbine trip. See LER #NP-33-84-13 for further details.
	84-09-14	S	408.0	C	9				The refueling outage was entered following the reactor trip on 9/11/84 to perform scheduled maintenance and refueling work.

¹
 F: Forced
 S: Scheduled

²
 Reason:
 A-Equipment Failure (Explain)
 B-Maintenance of Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

³
 Method:
 1-Manual
 2-Manual Scram.
 3-Automatic Scram.
 4-Continuation from Previous Month
 5-Load Reduction
 9-Other (Explain)

⁴
 Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

⁵
 Exhibit I - Same Source

(9/77)

OPERATING DATA REPORT

DOCKET NO. 50-346
 DATE 10-2-84
 COMPLETED BY Bilal M. Sarsour
 TELEPHONE (419)259-5000
 Ext. 384

OPERATING STATUS

1. Unit Name: Davis-Besse #1
2. Reporting Period: September, 1984
3. Licensed Thermal Power (MWt): 2772
4. Nameplate Rating (Gross MWe): 915
5. Design Electrical Rating (Net MWe): 906
6. Maximum Dependable Capacity (Gross MWe): 918
7. Maximum Dependable Capacity (Net MWe): 874
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

Notes

9. Power Level To Which Restricted, If Any (Net MWe): _____
10. Reasons For Restrictions, If Any: _____

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	<u>720</u>	<u>6,575.0</u>	<u>54,096.0</u>
12. Number Of Hours Reactor Was Critical	<u>252.6</u>	<u>5,529.0</u>	<u>33,031.5</u>
13. Reactor Reserve Shutdown Hours	<u>0.0</u>	<u>134.8</u>	<u>4,014.1</u>
14. Hours Generator On-Line	<u>252.6</u>	<u>5,489.5</u>	<u>31,641.3</u>
15. Unit Reserve Shutdown Hours	<u>0.0</u>	<u>0.0</u>	<u>1,732.5</u>
16. Gross Thermal Energy Generated (MWH)	<u>514,927</u>	<u>13,941,608</u>	<u>74,985,422</u>
17. Gross Electrical Energy Generated (MWH)	<u>163,161</u>	<u>4,554,151</u>	<u>24,846,344</u>
18. Net Electrical Energy Generated (MWH)	<u>148,183</u>	<u>4,291,557</u>	<u>23,290,256</u>
19. Unit Service Factor	<u>35.1</u>	<u>83.5</u>	<u>58.5</u>
20. Unit Availability Factor	<u>35.1</u>	<u>83.5</u>	<u>61.7</u>
21. Unit Capacity Factor (Using MDC Net)	<u>23.5</u>	<u>74.7</u>	<u>49.3</u>
22. Unit Capacity Factor (Using DER Net)	<u>22.7</u>	<u>72.0</u>	<u>47.5</u>
23. Unit Forced Outage Rate	<u>19.0</u>	<u>11.0</u>	<u>17.3</u>

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

25. If Shut Down At End Of Report Period, Estimated Date of Startup: _____

	Forecast	Achieved
INITIAL CRITICALITY	_____	_____
INITIAL ELECTRICITY	_____	_____
COMMERCIAL OPERATION	_____	_____

OPERATIONAL SUMMARY

SEPTEMBER, 1984

9/1/84 - 9/8/84

Reactor power was maintained at approximately 75% power with the generator gross load at approximately 662 ± 10 MWe until 1300 hours on September 8, 1984 when power was reduced to approximately 70% due to Cycle 4 coastdown.

9/9/84 - 9/30/84

Reactor Power was maintained at approximately 70% until 1235 hours on September 11, 1984 when a reactor trip occurred. The reactor tripped by the Anticipatory Reactor Trip System (ARTS) in response to the turbine trip which was caused by the inadvertent striking of the moisture separator reheater high level switch during pre-outage activities.

A plant cooldown was initiated to perform scheduled maintenance and refueling work. Full details on the work items performed during the scheduled maintenance outage will be presented in next month's Operational Summary.

REFUELING INFORMATION

DATE: September, 1984

1. Name of facility: Davis-Besse Unit 1
2. Scheduled date for next refueling shutdown: June, 1986
3. Scheduled date for restart following refueling: December 22 1984
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? If answer is yes, what in general will these be? If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?

Ans: Expect the Reload Report to require standard reload fuel design Technical Specification changes (3/4.1 Reactivity Control Systems and 3/4.2 Power Distribution Limits).

5. Scheduled date(s) for submitting proposed licensing action and supporting information: July, 1984
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.

Ans: None identified to date.

7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool.
(a) 177 (b) 140 - Spent Fuel Assemblies
8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies.

Present: 735 Increase size by: 0 (zero)

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.

Date: 1993 - assuming ability to unload the entire core into the spent fuel pool is maintained.

COMPLETED FACILITY CHANGE REQUEST

FCR NO: 84-039

SYSTEM: Auxiliary Feedwater System

COMPONENT: MV 05990 and MV 06080

CHANGE, TEST OR EXPERIMENT: This FCR was implemented to change the torque switch settings on MV 05990 and MV 06080 from 1.5 open/1.5 close to 1.5 open/1.0 closed. These values were taken from the Torrey Pines Technology Report. Work involved with FCR 84-039 was completed March 6, 1984.

REASON FOR CHANGE: MV 05990 torqued out while attempting to open during a recent shutdown. The new torque switch settings are taken from the Torrey Pines Technology Report in an effort to improve valve reliability.

SAFETY EVALUATION: The work authorized by this FCR does not create any new adverse environmental effect and does not constitute an unreviewed safety question.

COMPLETED FACILITY CHANGE REQUEST

FCR NO: 80-272

SYSTEM: N/A

COMPONENT: Containment Barricade

CHANGE, TEST OR EXPERIMENT: This FCR was implemented to install a barricade over the 3' high space around the plate of the incore monitor guide tubes on the 565' elevation in containment. Work was completed on September 8, 1983.

REASON FOR CHANGE: This change occurred due to the overexposure of personnel on April 30, 1980 as a result of inadequate barricades.

SAFETY EVALUATION: The barricade will result in a reduced possibility of inadvertent personnel radiation overexposure. Therefore, this FCR does not involve an unreviewed safety question.

COMPLETED FACILITY CHANGE REQUEST

FCR NO: 81-029

SYSTEM: N/A

COMPONENT: Concrete Masonry Block Wall No. 4036

CHANGE, TEST OR EXPERIMENT: A steel beam was installed on each side along the top of wall 4036 to brace the top connection and a steel angle. A beam has also been used near the top of the wall along each side to provide lateral bracing. Wall 4036 separates Electrical Penetration Room #1 (428B) from the Low Voltage Switchgear Room (428) at an elevation of 603'. Work was completed March 9, 1984.

REASON FOR CHANGE: Block wall reanalysis per NRC IE-Bulletin 80-11 has shown that during a seismic event the concrete masonry, the vertical reinforcing steel, and the east edge, bottom, and top connections of wall 4036 could become overstressed.

SAFETY EVALUATION: This modification has been reviewed and found not to adversely affect the ability of the wall to function as a fire barrier in any adverse manner. Therefore, this FCR does not involve an unreviewed safety question.

COMPLETED FACILITY CHANGE REQUEST

FCR NO: 80-047

SYSTEM: Reactor Coolant System

COMPONENT: Pipe Whip Restraint

CHANGE, TEST OR EXPERIMENT: FCR 80-047 allowed the modification of the upper hot leg pipe whip restraint shown on drawings 7749-C-176, Rev. 6 and 7749-C-600, Rev. 4 to meet the design criteria in the Final Safety Analysis Report. Modifications included the painting and priming of this piping, changing the bolt length to 12-3/4", and modifying the pipes installation. Work was completed February 3, 1981.

REASON FOR CHANGE: While performing the RCS pipe whip restraint verification study required by item 6.3 of the O.I.&E Inspection Report 50-346/79-23, it was determined that the upper hot leg pipe whip restraint did not meet the acceptance criteria.

SAFETY EVALUATION: Since the modified restraint will be able to perform its design function under all postulated conditions, an unreviewed safety question is not involved.



October 10, 1984

Log No. K84-1281
File: RR 2 (P-6-84-09)

Docket No. 50-346
License No. NPF-3

Mr. Norman Haller, Director
Office of Management and Program Analysis
U. S. Nuclear Regulatory Commission
Washington, D.C. 20555

Dear Mr. Haller:

Monthly Operating Report, September, 1984
Davis-Besse Nuclear Power Station Unit 1

Enclosed are ten copies of the Monthly Operating Report for Davis-Besse Nuclear Power Station Unit 1 for the month of September 1984.

If you have any questions, please feel free to contact Bilal Sarsour at (419) 239-5000, Extension 384.

Yours truly,

A handwritten signature in cursive script that reads "Stephen M. Quennoz".

Stephen M. Quennoz
Plant Manager
Davis-Besse Nuclear Power Station

SMQ/BMS/bec

Enclosures

cc: Mr. James G. Keppler, w/1
Regional Administrator, Region III

Mr. Richard DeYoung, Director, w/2
Office of Inspection and Enforcement

Mr. Walt Rogers, w/1
NRC Resident Inspector

LJK/002

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