NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
EDWIN I. HATCH, UNIT II	0 5 0 0 0 3 6 6	1 OF 0 3

TITLE (4)

RCIC Delta P Instrument FT&C Procedure Error

EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR		SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
0 1	2 4	8 5	8 5	—	0 0 6	—	0 2	2 2	8 5		0 5 0 0 0
											0 5 0 0 0

OPERATING MODE (9)	1	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)											
POWER LEVEL (10)	0 7 0		20.402(b)		20.405(c)		50.73(a)(2)(iv)		73.71(b)				
			20.405(a)(1)(i)		50.36(c)(1)		50.73(a)(2)(v)		73.71(c)				
			20.405(a)(1)(ii)		50.36(c)(2)		50.73(a)(2)(vii)		OTHER (Specify in Abstract below and in Text, NRC Form 366A)				
			20.405(a)(1)(iii)	X	50.73(a)(2)(i)		50.73(a)(2)(viii)(A)						
			20.405(a)(1)(iv)		50.73(a)(2)(ii)		50.73(a)(2)(viii)(B)						
			20.405(a)(1)(v)		50.73(a)(2)(iii)		50.73(a)(2)(x)						

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
	AREA CODE
T. L. Elton, Acting Superintendent of Regulatory Compliance	9 1 2 3 6 7 - 7 8 5 1

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 01-24-85, during performance of a biennial review of the "RCIC STEAM LINE DELTA P INSTRUMENT FT&C" procedure (HNP-2-3410), plant personnel determined that the procedure contained an error in the calibration of RCIC steam line differential pressure transmitters 2E51-N057A and 2E51-N057B. The procedure (HNP-2-3410) error resulted from an error where the transmitters' static head corrections were mistakenly applied to the incorrect side of each differential pressure transmitter.

An LCO was initiated and RCIC was isolated and declared inoperable at approximately 0950 CST on 01-24-85. The RCIC STEAM LINE DELTA P INSTRUMENT FT&C" procedure (HNP-2-3410) was then revised, and pressure transmitters 2E51-N057A and 2E51-N057B where recalibrated per the revision. RCIC was satisfactorily returned to service at approximately 2120 CST on 01-24-85. This event is reportable per 10 CFR 50.73(a)(2)(i) because the plant was operated (from 08-29-84 to 01-24-85) in a condition that is prohibited by Tech. Specs. section 3.3.2, TABLE 3.2.2-2, ITEM 5a. This event is the result of a defective procedure.

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TEXT (If more space is required, use additional NRC Form 366A's) (17)

This 30 day LER is required by 10 CFR 50.73(a)(2)(i), because this event shows that the plant was operated in a condition that is prohibited by Tech. Specs. section 3.3.2; TABLE 3.3.2-2, ITEM 5a.

On 01/24/85 (with the plant operating at 1692 MWt (approximately 70% power), during a biennial review of the "RCIC STEAM LINE DELTA P INSTRUMENT FT&C" procedure (HNP-2-3410) plant personnel determined that the calibration head correction data for RCIC steam line differential pressure transmitters 2E51-N057A and 2E51-N057B was incorrect, and the incorrect data had been incorporated into the functional test and calibration procedure (HNP-2-3410).

New RCIC steam line differential pressure transmitters (2E51-N057A and 2E51-N057B) were installed during the Unit 2 recirculation piping outage (1-14-84 to 8-29-84) per a Design Change Request (DCR) as part of the Analog Transmitter Trip system (ATTS) modification. After installation of the differential pressure transmitters, their spans were calculated (including static head correction) to input current into their respective trip units (2E51-N657A and 2E51-N657B) so that the trip units would isolate RCIC at equal to or less than 312% rated steam flow per Tech. Specs. Table 3.3.2-2, item 5a. However, the static head corrections were mistakenly applied to the incorrect side of each differential pressure transmitter (i.e., the calibration head correction data that should have applied to the low side of the transmitters was mistakenly applied to the transmitters' high side, and vice versa). As a result of the mistake, procedure HNP-2-3410, which was revised on 07-21-84, included the error. Consequently, when plant personnel used procedure HNP-2-3410 to initially calibrate the RCIC steam line differential pressure transmitters (2E51-N057A and 2E51-N057B) prior to Unit 2 startup, transmitter 2E51-N057A was erroneously calibrated on 08-14-84 so that it would have given an isolation signal at approximately 316% of rated steam flow instead of 312% as required by Tech. Specs. Differential pressure transmitter 2E51-N057B was calibrated on 08-15-84 so that it would have given an isolation signal at approximately 200% (conservative) of rated steam flow, instead of the required 312%.

When it was determined that procedure HNP-2-3410 was in error, an LCO was established at approximately 0950 CST on 01-24-85. At that time RCIC was declared inoperable and isolated per Tech. Specs. TABLE 3.3.2-1, ITEM 5a, ACTION 26. It should be noted that other RCIC steam line isolation instrumentation (differential temperature transmitter) was available which would have isolated RCIC in the event of a RCIC steam line break, and subsequent temperature rise.

This non-repetitive event had no actual or potential safety consequences or implications. The health and safety of the public were not affected by this event.

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TEXT (If more space is required, use additional NRC Form 366A's) (17)

This event is the result of a defective procedure. The "RCIC STEAM LINE DELTA P INSTRUMENT FT&C" procedure (HNP-2-3410) was then revised to reflect the correct calibration data. RCIC steam line differential pressure transmitters 2E51-N057A and 2E51-N057B were then satisfactorily re-calibrated per the procedure (HNP-2-3410) revision and returned to service on 01-24-85. The LCO was terminated at approximately 2120 CST on 01-24-85, and RCIC was declared operable at that time.

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Georgia Power

Edwin I. Hatch Nuclear Plant

February 22, 1985
GM-85-150

PLANT E. I. HATCH
Licensee Event Report
Docket No. 50-366

United States Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555

Attached is Licensee Event Report No. 50-366/1985-006. This report is required by 10CFR 50.73(a)(2)(i).

H. C. Nix
General Manager

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HCN/TLE/vlz

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