

DUKE POWER COMPANY

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VICE PRESIDENT
NUCLEAR PRODUCTION

TELEPHONE
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November 2, 1984

Mr. Harold R. Denton, Director
Office of Nuclear Reactor Regulation
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

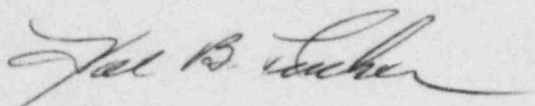
Attention: Ms. E. G. Adensam, Chief
Licensing Branch No. 4

Re: Catawba Nuclear Station
Docket Nos. 50-413 and 50-414

Dear Mr. Denton:

My letter of October 19, 1984 provided the status of specific commitments made in our response to Generic Letter 83-28 which was sent to you on November 4, 1983. Please find attached supplementary information outlining Duke Power Company's schedule for implementation of the requirements specified in Generic Letter 83-28.

Very truly yours,



Hal B. Tucker

ROE:slb

Attachment

cc: Mr. James P. O'Reilly, Regional Administrator
U. S. Nuclear Regulatory Commission
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CATAWBA NUCLEAR STATION

GENERIC LETTER 83-28

Status of Requirements

Item 1.1 Post Trip Review (Program Description and Review).

The Post Trip Review Program was described in our response, dated November 4, 1983. The program is in place and will be implemented upon initial criticality.

Item 1.2 Post Trip Review (Data and Information Capability).

The Post Trip Review data and information capability was described in our response, dated November 4, 1983. It is part of our Post Trip Review Program which will be implemented upon initial criticality.

Item 2.1 Equipment Classification and Vendor Interface (Reactor Trip System Components).

The Equipment Classification program and the Vendor Interface Program for Reactor Trip System Components were described in our response dated November 4, 1983. These programs have been implemented.

Item 2.2 Equipment Classification and Vendor Interface (All Safety-Related Components).

The Equipment Classification Program was described in our November 4, 1983 response and is currently being implemented. The Vendor Interface Programs for all Safety-Related components were described in our response of May 7, 1984. This program will be fully implemented prior to December 31, 1986.

Item 3.1 Post-Maintenance Testing (Reactor Trip System Components).

Our review of vendor technical information for Reactor Trip System Components is in progress. The results of this review will be submitted prior to December 31, 1984. Any Technical Specification changes will be submitted at that time.

The Post-Maintenance Testing Program for Reactor Trip System Components has been implemented.

Item 3.2 Post-Maintenance Testing (All other Safety-Related Components).

A review of vendor technical information is in progress. The results will be submitted by the end of the first refueling outage. Any Technical Specification changes will be submitted at that time.

The Post-Maintenance Testing Program has been implemented.

Item 4.1 Reactor Trip System Reliability (Vendor-Related Modifications).

Refer to our response dated November 4, 1983. This modification was completed prior to fuel load.

Items 4.2.1 Reactor Trip System Reliability (Preventative Maintenance and
and 4.2.2 Parameter Trending Program for Reactor Trip Breakers).

The Preventative Maintenance Program for Reactor Trip Breakers and the Parameter Trending program, if needed, will be implemented by December 31, 1984. A description of the Preventative Maintenance Program for Reactor Trip Breakers will be submitted by December 31, 1984.

Items 4.2.3 Reactor Trip System Reliability (Life Testing and Replacement
and 4.2.4 Program for Reactor Trip Breakers).

The Reactor Trip Breaker Trip Attachment Life Testing and Replacement programs will be submitted prior to startup from the first refueling outage. The Replacement program, which is a part of the Preventative Maintenance program, will be implemented by December 31, 1984.

Item 4.3 Reactor Trip System Reliability (Automatic Actuation of Shunt Trip Attachment for W and B&W Plants).

The Shunt trip attachment modification was described in our response of November 4, 1983 and completed prior to fuel load.

Item 4.5 Reactor Trip System Reliability (System Functional Testing).

We are awaiting NRC review of the Westinghouse Owner's Group WCAP-1011 - Supplement 1. Any changes to the surveillance requirements of the Standard Technical Specifications, will be incorporated into the Catawba Technical Specifications.

These changes will be submitted prior to startup from the first refueling outage.