NRC F (12-81 10 CF)		REGULATORY COMMISSION	APPROVED BY OMB
1001	CONTROL BLOCK: 1	(PLEASE PRINT OR TYPE ALL REQUIRED INFO	RMATION)
0 1	A L B R F 3 2 0 0 - 0 0 LICENSEE CODE 14	0 0 0 - 0 0 3 4 1 1 1 1 1 1 NUMBER 25 26 LICENSE TYPE 30	57 CAY 58 (5)
CON'T	NEPORT L 6 0 5 0 0 2 9	6 7 0 1 1 6 8 3 8 1 75 REPORT C	DATE 80
0 2	EVENT DESCRIPTION AND PROBABLE CONSEQUENCES    With unit 3 in maintenance outage,	the RHR 3D heat exchanger leaked	reactor
0 3	coolant into the RHR service water	, which discharges to Wheeler Rese	rvoir.
0 4	The activity level of the discharg	ed water was in excess of Technica	1
0 5	Specification (TS) 3.8.A.1 limits.	Redundant safety systems were in	itiated and
0 6	available. RHR 3D heat exchanger	was isolated. There was no danger	to health
0 7	or safety of the public, and no re	sulting significant chain of event	s.
0 8	SYSTEM CAUSE CAUSE	COMP. VALVE	80
0 9	C F 10 E 12 X 13	H TE X C H 4 G 5	9
,	SEQUENTIAL REPORT NO.	OCCURRENCE REPORT TYPE	REVISION NO.
	ACTION FUTURE EFFECT SHUTDOWN TAKEN ACTION ON PLANT METHOD HO	27 28 29 30 31  DURS 22 ATTACHMENT NPRO-4 PRIME COMP.  SUBMITTED FORM SUB. SUPPLIER  O O U Y 23 Y 24 N 25	32 COMPONENT 26 MANUFACTURER 26 P 1 6 0
1 0	CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)  Investigation disclosed twelve den	ted tubes in the heat exchanger, w	rith one
1 1	dented tube leaking. All twelve d	ented tubes were plugged. An inve	stigation
1 2	will continue during the next refu	el outage to determine the cause.	A follow-
1 3	up report will be submitted at tha	t time.	
1 4	8 9	METHOD OF	80
1 5	G 28 0 0 0 0 0 0 NA	A Radiation Alarm & Crab Sa	mple Analysis
,	ACTIVITY CONTENT AMOUNT OF ACTIVITY (35)    L  (33)   M (34)   0.0145 curies	From RHRSW to Wheeler Reser	
7	PERSONNEL EXPOSURES NUMBER TYPE DESCRIPTION (39)	45	80
1 7		NA PDR ADOCK 05000296	80
1 8	PERSONNEL INJURIES NUMBER DESCRIPTION 41	NA PDR	
7	S 9 11 12 LOSS OF OR DAMAGE TO FACILITY TYPE DESCRIPTION (43)	140	33 "
1 9	9 10	NA NA	80
20	Y 4 Information releases to the p	1 1111	C USE ONLY
	NAME OF PREPARER J. B. Walker		-3865

## LER SUPPLEMENTAL INFORMATION

BFRO-50- 296 / 8304	Technical Specification Involve	ed3.8.A.1
Reported Under Technical	Specification 6.7.2.b.(4) * Da	te Due NRC
Event Narrative:		- Duo IIIIO

Unit 1 was at 99 percent power and units 2 and 3 were in refueling and maintenance outages, respectively. Only unit 3 was affected by this event. During maintenance on unit 3, reactor coolant leaked from RHR heat exchanger 3D into the RHR service water system which discharges into the Wheeler Reservoir. The concentration in the discharge water released was in excess of Technical Specification 3.8.A.1 limits. The heat exchanger was immediately isolated. 3B RHR heat exchanger was available and placed in service. Because the radiation monitor monitoring 3B heat exchanger is also common to 3D heat exchanger, residual radioactivity caused an alarm with 3B heat exchanger in service. It was isolated, resulting in complete loss of shutdown cooling capability. (3A and 3C RHR was out of service previously for valve maintenance). This resulted in initiation of an ALERT in accordance with the Radiological Emergency Plan. Reactor cooling was maintained through the condenser, CRD and reactor water clean-up systems until it was confirmed that the suspected leak on 3B heat exchanger was due to the false indication. 3B heat exchanger was returned to service and the ALERT terminated. Both sides of 3D heat exchanger were isolated and drained to prevent further radioactive discharge from the plant.

(continued)

## \* Previous Similar Events: None

Retention: Period - Lifetime; Responsibility - Document Control Supervisor \*Revision:

## Event Narrative (Continued)

Corrective maintenance was initiated to locate and repair the leaking heat exchanger. Extensive analysis (see attached vendor report) revealed twelve tubes to be dented with one dented tube leaking. All 12 dented tubes were plugged.

Metallurgical examination of the subject tube has been performed. Leakage was through a circumferentially oriented crack. Mechanical damage to the tube was noted in the location of the crack. This mechanical damage appears to have caused the failure. "Bite" marks on either side of the crack are additional evidence of mechanical damage. Several metallographic cross sections of the crack did not reveal any evidence of corrosion assistance to the failure. It is most likely that this tube was installed in the heat exchanger in a damaged condition. Since the other tubes found to be in a mechanically damaged condition have been plugged, no further corrective actions are recommended at this time.

## TENNESSEE VALLEY AUTHORITY

P. O. Box 2000 Decatur, Alabama 35602

November 2, 1984

U. S. Nuclear Regulatory Commission Document Control Desk Washington, D. C. 20555

Dear Sir:

TENNESSEE VALLEY AUTHORITY - BROWNS FERRY NUCLEAR PLANT (BFN) UNIT 3 - DOCKET NO. 50-296 - FACILITY OPERATING LICENSE DPR-09 - REPORTABLE OCCURRENCE REPORT BFR0-50-296/83004 R1

The enclosed report provides additional details concerning a release into Wheeler Reservoir in excess of Technical Specification limits. This report is submitted in accordance with Browns Ferry Unit 3 Technical Specification 6.7.2.b.(4).

Very truly yours,

TENNESSEE VALLEY AUTHORITY

G. T. Jones
Plant Manager
Browns Ferry Nuclear Plant

Enclosure

cc (Enclosure):

Regional Administrator
U. S. Nuclear Regulatory Commission
Office of Inspection and Enforcement

Office of Inspection and Enforcement Region II 101 Marietta Street, Suite 2900

101 Marietta Street, Suite 2900 Atlanta, Georgia 30303

INPO Records Center Suite 1500 1100 Circle 75 Parkway Atlanta, Georgia 30339

NRC Resident Inspector, BFN

IE2211