

LICENSEE EVENT REPORT

EXHIBIT A

CONTROL BLOCK: _____ (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

7 0 1 | 8 A | R | A | N | 0 | 2 | 2 | 0 | 0 | 0 | 0 | 0 | 0 | 0 | 0 | 0 | 1 3 | 4 | 1 | 1 | 1 | 1 | 1 4 | 1 | 1 5
9 LICENSEE CODE 14 15 LICENSE NUMBER 25 26 LICENSE TYPE 30 57 CAT 58

7 0 1 | 8 R E P O R T | L 1 6 | 0 | 5 | 0 | 0 | 0 | 3 | 6 | 8 | 7 | 0 | 2 | 1 | 2 | 8 | 2 | 8 | 0 | 2 | 1 | 4 | 8 | 5 | 9
SOURCE 60 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES 10
On 02/12/82, 02/17/82, 03/19/82, and 03/29/82 while in Mode 1 at 100% full power, a containment isolation valve for the Reactor Building Sump (RBS) drain to the Auxiliary Building Sump, 2CV-2061-2, failed to fully close after draining the RBS. This occurrence caused entry into action b. of Tech. Spec. 3.6.3.1 and is reportable per T.S. 6.9.1.9.(b). Redundant isolation valve 2CV-2060-1 was operable.

7 0 2 | 8
7 0 3 |
7 0 4 |
7 0 5 |
7 0 6 |
7 0 7 |
7 0 8 |
7 9

SYSTEM CODE CAUSE CODE CAUSE SUBCODE COMPONENT CODE COMP VALVE
7 0 9 | 1 M | A | 1 1 | E | 1 2 | B | 1 3 | V | A | L | V | O | P | 1 4 | D | 1 5 | Z | 1 6
9 10 11 12 13 18 19 20

LER/RO EVENT YEAR SEQUENTIAL OCCURRENCE REPORT REVISION
17 REPORT NUMBER | 8 | 2 | 2 | 1 | 2 | 1 | 3 | 1 | X | 1 | NO
NUMBER | 21 | 22 | 23 | 24 | 25 | 26 | 27 | 28 | 29 | 30 | 31 | 32

ACTION FUTURE EFFECT SHUTDOWN ATTACHMENT NPRD-4 PRIME COMP. COMPONENT
TAKEN ACTION ON PLANT METHOD HOURS SUBMITTED FORM SUB SUPPLIER MANUFACTURER
7 0 1 8 | 1 2 | 1 9 | Z | 2 0 | Z | 2 1 | 0 | 0 | 0 | 0 | 2 2 | Y | 2 3 | Y | 2 4 | A | 2 5 | M | 1 | 3 | 8 | 2 6
33 34 35 36 37 40 41 42 43 44 47

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS 27
Immediate action was to close the redundant isolation valve, 2CV-2060-1 and deenergize its associated breaker pursuant to action b. of T.S. 3.6.3.1. Valve 2CV-2061-2 failed to close completely because the pneumatic actuator did not produce adequate closing force to overcome frictional forces. The actuator was disassembled, and no obvious mechanical problems were discovered. The seals were lubricated, and the actuator was re-assembled. The air supply line was relocated to a vertical tap on the actuator to (Continued)

FACILITY STATUS % POWER OTHER STATUS METHOD OF DISCOVERY DISCOVERY DESCRIPTION
7 1 5 | 8 E | 2 8 | 1 | 0 | 0 | 0 | 2 9 | N A | 3 0 | A | 3 1 | Operator Observation 132
9 10 12 13 44 45 46 50

ACTIVITY CONTENT AMOUNT OF ACTIVITY LOCATION OF RELEASE
7 1 6 | 8 Z | 1 3 3 | 2 | 1 3 4 | 1 | N A | 3 5 | 4 5 | 1 3 6
9 10 11 44 45 80

PERSONNEL EXPOSURES NUMBER TYPE DESCRIPTION
7 1 7 | 8 0 | 0 | 0 | 0 | 3 7 | Z | 1 3 8 | N A 139
9 11 12 13 80

PERSONNEL INJURIES NUMBER DESCRIPTION
7 1 8 | 8 0 | 0 | 0 | 0 | 4 0 | N A 141
9 11 12 80

LOSS OF OR DAMAGE TO FACILITY TYPE DESCRIPTION
7 1 9 | 8 Z | 1 4 2 | N A 143
9 143

PUBLICITY ISSUED DESCRIPTION
7 2 0 | 8 N | 1 4 4 | N A 145
9 10 68 69 80

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8503060416 850214
PDR ADOCK 05000368
S PDR

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ATTACHMENT

LER NO. 50-368/82-006/03X-2

Cause Description and Corrective Actions (Continued)

facilitate better lubrication of the actuator seals. The valve was tested several times, and isolation times were less than required by T.S. Table 3.6-1 thus proving operability. The valve was returned to service. A subsequent engineering evaluation resulted in replacement of the EPG ball valve and Matryx actuator with a Canadian Worchester Controls pneumatic actuated valve package during the 1983 refueling outage. 2CV-2060-1 was released to operations on 12/06/83 and operation has been satisfactory since that time.



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February 14, 1985

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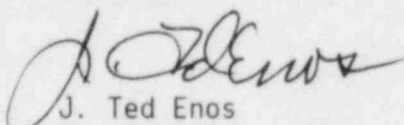
U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Subject: Arkansas Nuclear One - Unit 2
Docket No. 50-368
License No. NPF-6
Licensee Event Report
No. 82-006/03X-2

Gentlemen:

In accordance with Arkansas Nuclear One - Unit 2 Technical Specification 6.9.1.9.(b), attached is the subject report concerning a failure of containment isolation valve 2CV-2061-2. This is an update to a previous submittal dated April 16, 1982.

Very truly yours,



J. Ted Enos
Manager, Licensing

JTE:RJS:ds

Attachment

cc: Mr. Richard C. DeYoung
Office of Inspection and Enforcement
U. S. Nuclear Regulatory Commission
Washington, DC 20555

Mr. Norman M. Haller, Director
Office of Management & Program Analysis
U. S. Nuclear Regulatory Commission
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