NRC FORM 366
(7-77)

U.S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

EXHIBIT A

CONTROL BLOCK: | | | | | | | 1 ... 6 (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

| 0 | 1 | LICENSEE CODE (9–14): A R A N O 2 | 2 | LICENSE NUMBER (15–25): 0 0 - 0 0 0 0 0 - 0 0 | 3 | LICENSE TYPE (26–30): 4 1 1 1 1 | 4 | CAT (57–58): | 5

| 0 | 1 | REPORT SOURCE (60): L | 6 | DOCKET NUMBER (61–68): 0 5 0 0 0 3 6 8 | 7 | EVENT DATE (69–74): 0 2 1 2 8 2 | 8 | REPORT DATE (75–80): 0 2 1 4 8 5 | 9

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES 10

| 0 | 2 | On 02/12/82, 02/17/82, 03/19/82, and 03/29/82 while in Mode 1 at 100% full power, a containment isolation valve
| 0 | 3 | for the Reactor Building Sump (RBS) drain to the Auxiliary Building Sump, 2CV-2061-2, failed to fully close
| 0 | 4 | after draining the RBS. This occurrence caused entry into action b. of Tech. Spec. 3.6.3.1 and is reportable
| 0 | 5 | per T.S. 6.9.1.9.(b). Redundant isolation valve 2CV-2060-1 was operable.
0	6
0	7
0	8

	SYSTEM CODE	CAUSE CODE	CAUSE SUBCODE	COMPONENT CODE	COMP SUBCODE	VALVE SUBCODE
0 9	M A (11)	E (12)	B (13)	V A L V O P (14)	O (15)	Z (16)

LER/RO REPORT NUMBER (17)	EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE	REPORT TYPE		REVISION NO.
	8 2	---	0 0 6	/	0 3	X	---	2

ACTION TAKEN	FUTURE ACTION	EFFECT ON PLANT	SHUTDOWN METHOD	HOURS	ATTACHMENT SUBMITTED	NPRD-4 FORM SUB	PRIME COMP. SUPPLIER	COMPONENT MANUFACTURER
C (18)	Z (19)	Z (20)	Z (21)	0 0 0 0 (22)	Y (23)	Y (24)	A (25)	M 1 3 8 (26)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS 27

| 1 | 0 | Immediate action was to close the redundant isolation valve, 2CV-2060-1 and deenergize its associated breaker
| 1 | 1 | pursuant to action b. of T.S. 3.6.3.1. Valve 2CV-2061-2 failed to close completely because the pneumatic
| 1 | 2 | actuator did not produce adequate closing force to overcome frictional forces. The actuator was disassembled,
| 1 | 3 | and no obvious mechanical problems were discovered. The seals were lubricated, and the actuator was re-
| 1 | 4 | assembled. The air supply line was relocated to a vertical tap on the actuator to (Continued)

	FACILITY STATUS	% POWER	OTHER STATUS	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION
1 5	E (28)	1 0 0 (29)	NA (30)	A (31)	Operator Observation (32)

	ACTIVITY RELEASED	CONTENT OF RELEASE	AMOUNT OF ACTIVITY	LOCATION OF RELEASE
1 6	Z (33)	Z (34)	NA (35)	NA (36)

PERSONNEL EXPOSURES

	NUMBER	TYPE	DESCRIPTION
1 7	0 0 0 (37)	Z (38)	NA (39)

PERSONNEL INJURIES

	NUMBER	DESCRIPTION
1 8	0 0 0 (40)	NA (41)

LOSS OF OR DAMAGE TO FACILITY

	TYPE	DESCRIPTION
1 9	Z (42)	NA (43)

8503060416 850214
PDR ADOCK 05000368
S PDR

PUBLICITY

	ISSUED	DESCRIPTION	NRC USE ONLY
2 0	N (44)	NA (45)	

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IE22 1/1

LER NO. 50-368/82-006/03X-2

Cause Description and Corrective Actions (Continued)

facilitate better lubrication of the actuator seals. The valve was tested several times, and isolation times were less than required by T.S. Table 3.6-1 thus proving operability. The valve was returned to service. A subsequent engineering evaluation resulted in replacement of the EPG ball valve and Matryx actuator with a Canadian Worchester Controls pneumatic actuated valve package during the 1983 refueling outage. 2CV-2060-1 was released to operations on 12/06/83 and operation has been satisfactory since that time.

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February 14, 1985

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U. S. Nuclear Regulatory Commission
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Washington, D.C. 20555

Subject: Arkansas Nuclear One - Unit 2
Docket No. 50-368
License No. NPF-6
Licensee Event Report
No. 82-006/03X-2

Gentlemen:

In accordance with Arkansas Nuclear One - Unit 2 Technical Specification 6.9.1.9.(b), attached is the subject report concerning a failure of containment isolation valve 2CV-2061-2. This is an update to a previous submittal dated April 16, 1982.

Very truly yours,

J. Ted Enos
Manager, Licensing

JTE:RJS:ds

Attachment

cc: Mr. Richard C. DeYoung
Office of Inspection and Enforcement
U. S. Nuclear Regulatory Commission
Washington, DC 20555

Mr. Norman M. Haller, Director
Office of Management & Program Analysis
U. S. Nuclear Regulatory Commission
Washington, DC 20555

IE22
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