

LASALLE NUCLEAR POWER STATION

UNIT 1

MONTHLY PERFORMANCE REPORT

SEPTEMBER 1984

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-373

LICENSE NO. NPF-11

8411140455 840930
PDR ADOCK 05000373
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I. INTRODUCTION

The LaSalle Nuclear Power Station is a Two Unit Facility Located in Marseilles, Illinois. Each Unit is a Boiling Water Reactor with a designed electrical output of 1078 MWe net. The Station is owned by Commonwealth Edison Company. The Architect/Engineer was Sargent & Lundy, and the primary construction contractor was Commonwealth Edison Company.

The condenser cooling method is a closed cycle cooling pond. Unit One is subject to License Number NPF-11, issued on April 17, 1982. The date of initial criticality was June 21, 1982. Unit Two is subject to license number NPF-18, issued on December 16, 1983. The date of initial criticality was March 10, 1984.

This report was compiled by Randy S. Dus telephone number (815)357-6761, extension 324.

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II. MONTHLY REPORT FOR UNIT ONE

A. SUMMARY OF OPERATING EXPERIENCE FOR UNIT ONE

SEPTEMBER 1-22

The Unit started the reporting period at 83% power. At 0730 hours on September 3 power was reduced to 60% per the Load Dispatcher. At 1500 hours on September 4, reactor power was increased to 86%. At 0200 hours on September 5 power was increased to 86%. At 0200 hours on September 5 power was reduced to 75% per the Load Dispatcher. At 2300 hours on September 14 power was reduced to 55%. At 0700 hours on September 17, reactor power was 74%. At 0700 hours on September 19 power was reduced to 25% to troubleshoot the main generator voltage imbalance due to pot transformer fuses. At 1522 hours on September 19 the turbine generator was removed from the grid for OAD work. At 2040 hours on September 19 the turbine-generator was synchronized to the grid. At 1500 hours on September 20, reactor power was increased to 67%. At 1440 hours on September 21 the reactor scrambled due to performance of instrument surveillance, LIS-MS-01, Group I Isolation. The reactor was critical for 494 hours and 40 minutes.

SEPTEMBER 23-30

At 2000 hours on September 23, the reactor was critical. At 0505 hours on September 24, the main generator was synchronized to the grid. At 1030 hours on September 24, reactor power was increased to 25%. At 1500 hours on September 24, reactor power was increased to 52%. At 2300 hours on September 28, power was decreased to 36% for the outage. At 0345 hours on September 29 the generator was taken off line. At 1245 hours on September 29, the reactor mode switch was put in shutdown for the outage. The reactor was critical for 136 hours and 45 minutes.

B. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY RELATED MAINTENANCE.

1. Amendments to facility license or Technical Specification.

There were no facility license or Technical Specification Amendments during the reporting period.

2. Facility or procedure changes requiring NRC approval.

There were no facility or procedure changes requiring NRC approval.

3. Tests and Experiments requiring NRC approval.

There were no tests or experiments requiring NRC approval.

4. Corrective maintenance of safety related equipment.

The following table (Table 1) presents a summary of safety-related maintenance completed on Unit One during the reporting period. The headings indicated in this summary include: Work Request numbers, LER numbers, Component Name, Cause of Malfunction, Results and Effects on Safe Operation, and Corrective Action.

TABLE 1

CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENTLTP-300-7
Revision 4
February 29, 1984
6

WORK REQUEST	LER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE OPERATION	CORRECTIVE ACTION
L35938		Drywell/Sup pression pool temperature	Points 1 & 2 currently in an alarm state due to low alarm setpoint.	Drywell and suppression pool temperatures are monitored at all times.	Recalibrated alarm points 1 & 2 to come up @ 50°F.
L36087		RHR heat exch anger vent to suppression pool valve.	Valve does not fully close with handswitch in closed position.	Valve handwheel must be cranked an additional 1/2 turn to stop flow completely	Readjusted torque switch setting.
L37095		HPCS Full flow test valve to cy tank	Stake the setscrew in the Anti-rotation stem clamp to prevent loosening due to vibrations.	Preventative maintenance measure.	Removed, cleaned, lubricated setscrew with locktite and reinstalled.
L38320		Suppression chamber water temperature recorder	PT #13 reads downscale	Suppression pool water temperature monitored at all times via redundant channels	Replaced defective wire and connections.
L39495		2B D/G	The Starters for the diesel generator need to be changed out.	Technical specification 3.8.1 limits the number of starts for each starter.	Replaced air start motors.
L40480		Suppression pool narrow range re- order	Recorder printing false suppression pool level indications.	Pool level verified to be within limits via local sight glasses.	Recalibrated narrow range portion of LIS-CM-01.
L40709		HPCS Relief valve	Relief valve leaks badly	Potential loss of containment integrity	Removed valve, rebuilt and sat- isfactorily leak tested.
L40896		Main Steam line drain	Valve does not seal in closed from the control switch.	Valve still fully closes when handswitch is held closed.	Cleaned torque switch contacts.

C. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for LaSalle Nuclear Power Station, Unit One, occurring during the reporting period, September 1 through September 30, 1984. This information is provided pursuant to the reportable occurrence reporting requirements as set forth in 10CFR 50.73.

<u>Licensee Event Report Number</u>	<u>Date</u>	<u>Title of Occurrence</u>
84-047-00	8/7/84	RWCU High Differential Flow Isolation
84-048-00	8/10/84	Missed Noble Gas Sample from U-1 SBT & Particulate & Iodine Samples Counted Late
84-049-00	8/24/84	VR Isolation During Testing LES-RP-102
84-050-00	8/28/84	RWCU Isolation on High Delta Flow
84-051-00	8/29/84	SRV's Lifting

D. DATA TABULATIONS

The following data tabulations are presented in this report:

1. Operating Data Report
2. Average Daily Unit Power Level
3. Unit Shutdowns and Power Reductions

1. OPERATING DATA REPORT

DOCKET NO. 050-373
 UNIT LaSalle One
 DATE October 10, 1984
 COMPLETED BY Randy S. Dus
 TELEPHONE (815)357-6761

OPERATING STATUS

1. REPORTING PERIOD: September 1984 GROSS HOURS IN REPORTING PERIOD: 720
2. CURRENTLY AUTHORIZED POWER LEVEL (Mwt):3323 MAX DEPEND CAPACITY (MWe-Net): 1036 DESIGN ELECTRICAL RATING (MWe-Net):1078
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A
4. REASONS FOR RESTRICTION (IF ANY):

	THIS MONTH	YR TO DATE	CUMULATIVE
5. NUMBER OF HOURS REACTOR WAS CRITICAL	<u>631.4</u>	<u>5377.4</u>	<u>5377.4</u>
6. REACTOR RESERVE SHUTDOWN HOURS	<u>88.6</u>	<u>1165</u>	<u>1165</u>
7. HOURS GENERATOR ON LINE	<u>608.0</u>	<u>5194</u>	<u>5194</u>
8. UNIT RESERVE SHUTDOWN HOURS	<u>0.0</u>	<u>1.0</u>	<u>1.0</u>
9. GROSS THERMAL ENERGY GENERATED (MWH)	<u>1455185</u>	<u>14581709</u>	<u>14581709</u>
10. GROSS ELEC. ENERGY GENERATED (MWH)	<u>456560</u>	<u>4739789</u>	<u>4739789</u>
11. NET ELEC. ENERGY GENERATED (MWH)	<u>432697</u>	<u>4512808</u>	<u>4512808</u>
12. REACTOR SERVICE FACTOR	<u>87.7%</u>	<u>81.8%</u>	<u>81.8%</u>
13. REACTOR AVAILABILITY FACTOR	<u>100%</u>	<u>99.5%</u>	<u>99.5%</u>
14. UNIT SERVICE FACTOR	<u>84.4%</u>	<u>79.0%</u>	<u>79.0%</u>
15. UNIT AVAILABILITY FACTOR	<u>84.4%</u>	<u>79.0%</u>	<u>79.0%</u>
16. UNIT CAPACITY FACTOR (USING MDC)	<u>58.0%</u>	<u>66.2%</u>	<u>66.2%</u>
17. UNIT CAPACITY FACTOR(USING DESIGN MWe)	<u>55.7%</u>	<u>63.7%</u>	<u>63.7%</u>
18. UNIT FORCED OUTAGE RATE	<u>10.0%</u>	<u>17.1%</u>	<u>17.1%</u>
19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH)			
20. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: <u>November 1, 1984</u>			
21. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION):			

	FORECAST	ACHIEVED
INITIAL CRITICALITY	<u> </u>	<u>6/21/82</u>
INITIAL ELECTRICITY	<u> </u>	<u>9/04/82</u>
COMMERCIAL OPERATION	<u> </u>	<u>1/1/84</u>

2. AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO: 050-373
UNIT: LASALLE ONE
DATE: OCTOBER 10, 1984
COMPLETED BY: Randy S. Dus
TELEPHONE: (815) 357-6761

MONTH: SEPTEMBER 1984

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

1. _____	878	17. _____	767
2. _____	958	18. _____	811
3. _____	628	19. _____	174
4. _____	811	20. _____	572
5. _____	798	21. _____	265
6. _____	748	22. _____	0
7. _____	895	23. _____	0
8. _____	960	24. _____	271
9. _____	883	25. _____	654
10. _____	869	26. _____	562
11. _____	871	27. _____	564
12. _____	837	28. _____	527
13. _____	850	29. _____	6
14. _____	846	30. _____	0
15. _____	517	31. _____	0
16. _____	557		

INSTRUCTIONS

On this form list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt. These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line.) In such cases the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

ATTACHMENT E

3. UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 050-374
 UNIT NAME LaSalle One
 DATE October 1984
 COMPLETED BY Randy S. Dus
 TELEPHONE (815)357-6761

REPORT MONTH SEPTEMBER 1984

NO.	DATE	TYPE		DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS
		F: FORCED	S: SCHEDULED				
16	84/09/03		S	0.0	H	5	Reduced power per Load Dispatcher
17	84/09/14		S	0.0	H	5	Reduced power per Load Dispatcher
18	84/09/19	F		5.3	B	1	Reduced power to troubleshoot main generator voltage imbalance due to pot transformer fuses
19	84/09/21	F		62.4	B	3	Reactor scram caused by IM Surveillance LIS-MS-01.
20	84/09/29		S	44.3	H	1	Turbine off line for scheduled outage.

E. UNIQUE REPORTING REQUIREMENTS

1. Safety/Relief valve operations for Unit One.

<u>DATE</u>	<u>VALVES ACTUATED</u>	<u>NO & TYPE ACTUATION</u>	<u>PLANT CONDITION</u>	<u>DESCRIPTION OF EVENT</u>
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There were no relief valve actuations during the month of September.

2. ECCS Systems Outages

The following outages were taken on ECCS Systems during the reporting period.

<u>OUTAGE NO.</u>	<u>EQUIPMENT</u>	<u>PURPOSE OF OUTAGE</u>
1-680-84	1E22-F023	Prevent Rupture of discharge relief diaphragm.
1-699-84	1B D/G Motor Driven Air Compressor	Replace Head Gasket
1-731-84	1C RHR pump	Remove Motor
1-736-84	1C RHR Pump	Blank Flange Pump Suction Line
1-741-84	1C RHR Pump	Change Pump Shaft
1-747-84	1B D/G	18 Month Inspection
1-754-84	1E12-F009	Remove Motor
1-755-84	1E12-F008	Repairs to 1E12-F009
1-757-84	1E12-F009	Repair Limitorque
1-758-84	1E22-F028	ASTM UT-1 Visual Inspection
1-764-84	1B D/G	Recalibrate Instrumentation

3. Off-Site Dose Calculation Manual

There were no changes to the off-site dose calculations manual during this reporting period.

4. Radioactive Waste Treatment Systems.

There were no changes to the Radioactive Waste Treatment System during this reporting period.

LASALLE NUCLEAR POWER STATION

UNIT 2

MONTHLY PERFORMANCE REPORT

SEPTEMBER 1984

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-374

LICENSE NO. NPF-18

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 - 3. Off-Site Dose Calculation Manual Changes
 - 4. Major Changes to Radioactive Waste Treatment System

I. INTRODUCTION

The LaSalle Nuclear Power Station is a Two Unit Facility Located in Marseilles, Illinois. Each Unit is a Boiling Water Reactor with a designed electrical output of 1078 MWe net. The Station is owned by Commonwealth Edison Company. The Architect/Engineer was Sargent & Lundy, and the primary construction contractor was Commonwealth Edison Company.

The condenser cooling method is a closed cycle cooling pond. Unit One is subject to License Number NPF-11, issued on April 17, 1982. The unit commenced commercial generation of power on January 1, 1984. Unit Two is subject to license number NPF-18, issued on December 16, 1983. The date of initial criticality was March 10, 1984.

This report was compiled by Randy S. Dus, telephone number (815)357-6761, extension 324.

II. MONTHLY REPORT FOR UNIT TWO

A. SUMMARY OF OPERATING EXPERIENCE FOR UNIT TWO

September 1-8

The Unit started the reporting period at 70% power. At 2200 hours on September 3, reactor power was increased to 95%. At 0700 hours on September 5 power was reduced to 54% for STP-30, Reactor Recirculation Pump Trip. At 0000 hours on September 6, reactor power was increased to 91%. At 0515 hours on September 6, the reactor scrammed due to STP-25, MSIV closure. The reactor was critical for 125 hours and 15 minutes.

September 9-13

The reactor went critical at 0040 hours on September 9. At 1312 hours on September 9, the main generator was synchronized to the grid. At 1845 hours on September 9, reactor power was increased to 25%. At 2300 hours on September 9, reactor power was increased to 66%. At 2200 hours on September 12, reactor power was increased to 95%. At 2300 hours on September 12, the reactor scrammed due to STP 27-2, Turbine Control Valve Fast Closure. The reactor was critical for 94 hours and 20 minutes.

September 14-30

The reactor went critical at 0800 hours on September 14. At 1810 hours on September 14, the main generator was synchronized to the grid. At 2300 hours on September 14, reactor power was increased to 25%. At 0700 hours on September 15, reactor power was increased to 50%. At 0000 hours on September 16, reactor power was increased to 75%. At 1400 hours on September 17, reactor power was increased to 90%. At 0540 hours on September 18, reactor power was increased to 100%. At 1540 hours on September 25, power was reduced to 75% due to FCV operation without LVDT feedback. At 0156 hours on September 27, the turbine tripped on high vibration, reactor power was reduced to 20%. At 1900 hours on September 27, the main generator was synchronized to the grid. At 2200 hours on September 27, reactor power was increased to 42%. At 0700 hours on September 28, reactor power was increased to 65%. At 1400 hours on September 29, reactor power was increased to 88%. The reactor was critical for 400 hours.

B. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY RELATED MAINTENANCE.

1. Amendments to facility license or Technical Specifications.

Amendment No.4- This Amendment vacates Amendment No.3 and Reinstates License Condition 2.c.(7).

2. Facility or procedure changes requiring NRC approval.

There were no facility or procedure changes requiring NRC approval during the reporting period.

3. Tests and experiments requiring NRC approval.

There were no tests or experiments requiring NRC approval during the reporting period.

4. Corrective Maintenance of Safety Related Equipment.

The following table (Table 1) presents a summary of safety-related maintenance completed on Unit One during the reporting period. The headings indicated in this summary include: Work Request numbers, LER Numbers, Component Name, cause of malfunction, results and effects on safe operation, and corrective action.

TABLE 1
CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

LTP-300-7
Revision 4
February 29, 1984
6

WORK REQUEST	LER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE OPERATION	CORRECTIVE ACTION
L19912		Post LOCA H ₂ & O ₂ monitor ann.	Annunciators Do Not conform to "Black Panel" concept.	System still operational.	Rewired alarm logic to correct per modification package.
L28462		ITT Actuators for ventilation dampers	Fuses in control wiring logic too small to support full starting current.	Only a problem whenever 3 actuators are in parallel.	Replaced existing 6 amp fuses with 10 amp fuses per modification package.
L37864		VR Exhaust damper 1VR05YB	Damper shows dual indication. Discovered during Surveillance LOS-CS-Q1.	None Damper physically verified to be closed.	Adjusted limit switch to show closed indication.
L38689		2A D/G	Lubricating Oil Samples showed signs of fuel oil	Lubricating oil still within acceptable limits.	Tightened all fuel line connections. Resampled oil.
L39495		2B D/GT	The starters for the diesel generator need to be changed out.	Technical Specification 3.8.1 limits the number of starts for each starter	Replaced air start motors.
L39898		Div I Post Loca O ₂ recorder	Recorder pen cycling back and forth resulting in false indication.	Local O ₂ indication still showed acceptable O ₂ concentration.	Changed out servo drive gears and recalibrated.

TABLE 1

CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENTLTP-300-7
Revision 4
February 29, 1984
6

WORK REQUEST	LER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE OPERATION	CORRECTIVE ACTION
40472		Drywell temperature recorder	Recorder frequently stops driving.	Temperature indication still correct but recorder does not drive. Redundant channell still operable.	Replaced defective drive clutch.
L40872		VY Fan motor	Fan motor bearings are worn and need re- placement.	No effect on safe operation	Replaced motor bearings.
L40980		1B D/G motor driven air compressor	Headgasket blown so unable to maintain air reciever pressure greater than 220 psig.	Unable to air start the 1B	Installed new gasket and verified com- pressor operability.

C. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for LaSalle Nuclear Power Station, Unit Two, occurring during the reporting period, September 1 through September 30, 1984. This information is provided pursuant to the reportable occurrence reporting requirements as set forth in 10CFR 50.73.

<u>Licensee Event Report Number</u>	<u>Date</u>	<u>Title of Occurrence</u>
84-050-00	8/10/84	Reactor Scram from RPV High Pressure.
84-051-00	8/23/84	RWCU High Ambient Temperature Isolation.
84-052-00	8/17/84	Reactor Scram due to Turbine Trip.
84-053-00	8/16/84	Missed Off Gas H ₂ Sample.
84-054-00	8/18/84	RWCU Isolation on High Differential Flow.
84-055-00	8/18/84	Failure of 'B' RHR Full Flow Test to Close.
84-056-00	8/27/84	RWCU Isolation on High Pump Room Vent Differential Temperature.
84-057-00	9/6/84	Spurious RWCU High Differential Flow Isolation.
84-058-00	8/26/84	RWCU Room High Delta Temperature Isolation.
84-059-00	8/28/84	LIS-RD-403 Monthly Functional Missed Two consecutive Times.
84-060-00	8/28/84	CRD Charging Water Header Pressure Time Delay Greater Than Ten Seconds.
84-061-00	8/30/84	RWCU High Differential Flow Isolation.
84-062-00	8/27/84	Division II Isolation on RHR Shutdown Cooling.

D. DATA TABULATIONS

The following data tabulations are presented in this report:

1. Operating Data Report
2. Average Daily Unit Power Level
3. Unit Shutdowns and Power Reductions

1. OPERATING DATA REPORT

DOCKET NO. 050-374
 UNIT LaSalle Two
 DATE October 10, 1984
 COMPLETED BY Randy S. Dus
 TELEPHONE (815)357-6761

OPERATING STATUS

1. REPORTING PERIOD: September 1984 GROSS HOURS IN REPORTING PERIOD: 720
 2. CURRENTLY AUTHORIZED POWER LEVEL (Mwt): 3323 MAX DEPEND CAPACITY (MWe-Net): 1036 DESIGN ELECTRICAL RATING (MWe-Net): 1078
 3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A
 4. REASONS FOR RESTRICTION (IF ANY):
- | | THIS MONTH | YR TO DATE | CUMULATIVE |
|---|----------------|----------------|----------------|
| 5. NUMBER OF HOURS REACTOR WAS CRITICAL | <u>619.6</u> | <u>3430.6</u> | <u>3430.6</u> |
| 5. REACTOR RESERVE SHUTDOWN HOURS | <u>100.4</u> | <u>1480</u> | <u>1480</u> |
| 7. HOURS GENERATOR ON LINE | <u>579.8</u> | <u>2521.8</u> | <u>2521.8</u> |
| 8. UNIT RESERVE SHUTDOWN HOURS | <u>0.0</u> | <u>0.0</u> | <u>0.0</u> |
| 9. GROSS THERMAL ENERGY GENERATED (MWH) | <u>1515746</u> | <u>4804824</u> | <u>4804824</u> |
| 10. GROSS ELEC. ENERGY GENERATED (MWH) | <u>502128</u> | <u>1417452</u> | <u>1417452</u> |
| 11. NET ELEC. ENERGY GENERATED (MWH) | <u>482895</u> | <u>1343615</u> | <u>1343615</u> |
| 12. REACTOR SERVICE FACTOR | <u>N/A</u> | <u>N/A</u> | <u>N/A</u> |
| 13. REACTOR AVAILABILITY FACTOR | <u>N/A</u> | <u>N/A</u> | <u>N/A</u> |
| 14. UNIT SERVICE FACTOR | <u>N/A</u> | <u>N/A</u> | <u>N/A</u> |
| 15. UNIT AVAILABILITY FACTOR | <u>N/A</u> | <u>N/A</u> | <u>N/A</u> |
| 16. UNIT CAPACITY FACTOR (USING MDC) | <u>N/A</u> | <u>N/A</u> | <u>N/A</u> |
| 17. UNIT CAPACITY FACTOR (USING DESIGN MWe) | <u>N/A</u> | <u>N/A</u> | <u>N/A</u> |
| 18. UNIT FORCED OUTAGE RATE | <u>N/A</u> | <u>N/A</u> | <u>N/A</u> |
19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH)
 20. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP N/A
 21. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION):
- | | FORECAST | ACHIEVED |
|----------------------|-------------------|-------------------|
| INITIAL CRITICALITY | <u> </u> | <u>3/10/84</u> |
| INITIAL ELECTRICITY | <u> </u> | <u>4/20/84</u> |
| COMMERCIAL OPERATION | <u>Oct. 84</u> | <u> </u> |

2. AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO: 050-374
 UNIT: LASALLE TWO
 DATE: October 1984
 COMPLETED BY: Randy S. Dus
 TELEPHONE: (815) 357-6761
 MONTH: September 1984

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

1. _____	818	17. _____	927
2. _____	831	18. _____	1050
3. _____	935	19. _____	1047
4. _____	1005	20. _____	1044
5. _____	820	21. _____	1040
6. _____	215	22. _____	1042
7. _____	0	23. _____	1041
8. _____	0	24. _____	1036
9. _____	116	25. _____	959
10. _____	725	26. _____	779
11. _____	844	27. _____	62
12. _____	0	28. _____	688
13. _____	0	29. _____	920
14. _____	36	30. _____	872
15. _____	536	31. _____	
16. _____	733		

INSTRUCTIONS

On this form list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt. These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line.) In such cases the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

ATTACHMENT E

3. UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 050-374
 UNIT NAME LaSalle Two
 DATE October 1984
 COMPLETED BY Randy S. Dus
 TELEPHONE (815)357-6761

REPORT MONTH SEPTEMBER 1984

NO.	DATE	TYPE		DURATION (HOURS)	REASON (1)	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS
		F: FORCED	S: SCHEDULED				
28	840905	S		0.0	B	5	Reduced power for STP-30, Reactor Recirculation Pump Trip
29	840906	S		80.0	B	3	Reactor Scram for STP-25 MSIV Closure
30	840912	S		43.2	B	3	Reactor Scram for STP 27-2 Turbine Control Valve Fast Closure.
31	840925	F		0.0	A	5	Reduced power due to FCV operation w/o LVDT Feedback.
32	840927	F		17.0	A	9	Turbine Trip on high vibration.

E. UNIQUE REPORTING REQUIREMENTS

1. Safety/Relief Valve Operations for Unit Two.

<u>DATE</u>	<u>VALVES ACTUATED</u>	<u>NO & TYPE ACTUATIONS</u>	<u>PLANT CONDITION</u>	<u>DESCRIPTION OF EVENT</u>
9/6/84	2B21-F0135	1 Auto	90% Power	STP-25/RX Scram
9/6/84	2B21-F013U	1 Auto	90% Power	STP-25/RX Scram
9/6/84	2B21-F013D	1 Auto	90% Power	STP-25/RX Scram
9/11/84	2B21-F013E	1 Auto	95% Power	STP-27-2/RX Scram
9/11/84	2B21-F013S	1 Auto	95% Power	STP-27-2/RX Scram
9/11/84	2B21-F013D	1 Auto	95% Power	STP-27-2/RX Scram
9/11/84	2B21-F013U	1 Auto	95% Power	STP-27-2/RX Scram

2. ECCS Systems Outages

The following outages were taken on ECCS Systems during the reporting period.

<u>OUTAGE NO.</u>	<u>EQUIPMENT</u>	<u>PURPOSE OF OUTAGE</u>
2-1087-84	2E12-F041A	Repair Operator
2-1090-84	2E12-F003B	Repair Operator
2-1092-84	2E12-F074A	Tech Spec Action Statement
2-1098-84	2E12-F003B	Set Limit Switch
2-1103-84	2B D/G	Replace Air Start Motor
2-1113-84	2E22-F023	Maintenance on 2E22-F035.
2-1117-84	HPCS System	Install cy spool piece.
2-1123-84	LPCS Pump	Replace Lower Guide Bearing Thermocouple
2-1129-84	2B D/G	Recalibrate Instrumentation
2-1142-84	2B D/G Immersion Heater	Replace Control Power Transformer
2-1153-84	2B D/G Motor Driver Air compressor	Clean Moisture Trap
2-1174-84	HPCS System	Relief Valve Inspection
2-1177-84	HPCS Relief Valve	Repair Relief Valve

3. Off-Site Dose Calculation Manual

There were no changes to the off-site dose calculations manual during this reporting period.

4. Radioactive Waste Treatment Systems.

There were no changes to the Radioactive Waste Treatment System during this reporting period.

October 10, 1984

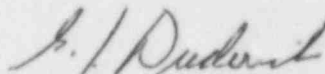
Director, Office of Management Information
and Program Control
United States Nuclear Regulatory Commission
Washington, D.C. 20555

ATTN: Document Control Desk

Gentlemen:

Enclosed for your information is the monthly performance report covering LaSalle County Nuclear Power Station for the period covering September 1 through September 30, 1984.

Very truly yours,



G. J. Diederich
Superintendent
LaSalle County Station

GJD/RSD/crh

Enclosure

xc: J. G. Keppler, NRC, Region III
NRC Resident Inspector LaSalle
Gary Wright, Ill. Dept. of Nuclear Safety
D. P. Galle, CECO
D. L. Farrar, CECO
INPO Records Center
Ron A. Johnson, PIP Coordinator SNED
W. R. Jackson, GE Resident
J. M. Nowicki, Asst. Comptroller