TENNESSEE VALLEY AUTHORITY DIVISION OF NUCLEAR POWER SEQUOYAH NUCLEAR PLANT

MONTHLY OPERATING REPORT

TO THE

NUCLEAR REGULATORY COMMISSION SEPTEMBER 1, 1984 - SEPTEMBER 30, 1984

> UNIT 1 DOCKET NUMBER 50-327 LICENSE NUMBER DPR-77

> > UNIT 2

DOCKET NUMBER 50-328 LICENSE NUMBER DPR-79

Submitted By:

D.R.

Plant Manager

TECH

8411140440 840930 PDR ADOCK 05000327 R PDR

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Operations Summary

September, 1984

The following summary describes the significant operational activities for the month of September. In support of this summary, a chronological log of significant events is included in this report.

Unit 1

Unit 1 was critical for 602.5 hours, produced 573,870 MWH (gross), resulting in an average hourly gross load of 1,017,951 kW during the month. There are 262.2 full power days estimated remaining until the end of cycle 3 fuel. With a capacity factor of 85 percent, the target EOC exposure would be reached August 6, 1985. The capacity factor for the month was 67.4 percent.

There were no reactor scrams, and one manual shutdown, and one power reduction during the month.

Unit 2

Unit 2 was critical for 575.05 hours, produced 482,390 MWH (gross), resulting in an average hourly gross load of 906,493 kW during the month. The cycle 2 refueling outage began September 28, 1984. The capacity factor for the month was 56.6 percent.

There were six reactor scrams, one manual shutdown, and no power reductions during the month.

Significant Operational Events

Unit 1

Date	Time	Event
08/01/84	0001	The reactor was in mode 1 at 99% power producing 1088 MWe. The investigation continued to determine the lost generation.
	0645	The cold leg accumulators were declared out of specification. Began reducing power for a manual shutdown.
	0703	The cold leg accumulators were declared in specification. Began power ascension.
	0920	The reactor obtained 100% power.

Unit 1

(Continued)

Date	Time	Event
09/19/84	1840	Power was interrupted to the UHI valves when the power cables were cut by a core drilling crew. Began reducing power for a manual shutdown.
	1922	The reactor was at 71% power and power was restored to the UHI valves. Began power ascension.
	2142	The reactor obtained 100% power.
09-21-84	2000	Began reducing power to remove the unit from service to repair the leaking feedwater drain line isolation valves.
	2339	The reactor entered mode 2.
	2343	The turbine was tripped.
09/22/84	0800	The reactor entered mode 3 and was still critical.
	1030	The shutdown banks were inserted.
	1520	The reactor entered mode 4.
	1900	The reactor entered mode 5.
09/26/84	0412	The reactor entered mode 4.
	1930	The reactor entered mode 3.
09/27/84	1447	The reactor was taken critical, entered mode 2.
09/28/84	1018	The reactor entered mode 1.
	1158	The turbine was tied on-line.
	1315	The reactor was in mode 1 at 30% power producing 282 MWe. Held due to steam generator chemistry.
09/30/84	2040	Began power ascension.
	2359	The reactor was in mode 1 at 69% power and increasing. The unit load was 795 MWe.

Unit 2

Date	Time	Event
09/01/84	0001	The reactor was in mode 1 at 30% power producing 288 MWe and was holding due to steam generator chemistry.
	1007	Began power ascension.
	2010	Held 75% reactor power awaiting repairs on CBP 2A.
	2050	2A CBP repairs were completed. Resumed power ascension.
09/02/84	0209	The reactor obtained 100% power.
09/04/84	0532	The turbine tripped due to low EHC pressure. A reactor trip followed.
09/05/84	2210	The reactor was taken critical.
	2245	The reactor was at 4% power and holding due to chemistry.
09/06/84	0822	Began power ascension.
	1000	The reactor was in mode 1 at 24% power. The turbine tripped followed by a reactor trip due to a Lo-Lo level in #4 steam generator.
09/06/84	1215	The reactor was taken critical.
	1333	The reactor entered mode 1, the turbine was not on-line.
	1416	The reactor was at 12% power when the turbine tripped due to fluctuation in the EHC system.
	1535	A Hi-Hi steam generator level tripped the turbine, the reactor was at 16% power and the unit was not on-line.
	1609	The unit was tied on-line.
	1745	The reactor was at 30% power producing 245 MWe.

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Unit 2

(Continued)

Date	Time	Event
09/08/84	0001	Began power ascension.
	2110	The reactor obtained 100% power, producing 1132 MWe.
09/09/84	0803	The turbine tripped followed by a reactor trip due to a neutral transformer overvoltage indication.
09/10/84	0240	The reactor was taken critical.
	0412	The reactor tripped due to a Lo-Lo steam generator level. The unit was not on-line.
	0535	The reactor was taken critical.
	0625	The reactor entered mode 1.
	0811	The unit was tied on-line.
	0859	The turbine tripped due to a Hi-Hi steam generator level.
	0901	The reactor was maintaining 30% power when it tripped due to a Lo-Lo steam generator level.
	1038	The reactor was taken critical.
	1138	The reactor entered mode 1.
	1217	The unit was tied on-line.
09/10/84	1253	The reactor tripped due to a Hi-Hi #4 steam generator level.
	1255	The reactor was at 30% power when a Lo-Lo #2 steam generator level caused it to trip.
	1433	Rod P-4 in shutdown bank A was declared inoperable. The unit began cool down to mode 5.
09/11/84	0157	The reactor entered mode 4.
	0530	The :eactor entered mode 5.
	1841	Rod P-4 was declared operable.

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Unit 2

(Continued)

Date	Time	Event
09/12/84	0313	The reactor entered mode 4.
	1018	The reactor entered mode 3.
	2012	The reactor was taken critical.
09/13/84	1000	The reactor was in mode 2 at 4% power and was holding due to chemistry and repairs to loop 2 feedwater reg valves.
	1340	Began power ascension.
	1448	The reactor entered mode 1.
	1726	The unit was tied on-line.
	2155	The reactor was at 30% power, producing 273 MWe and was holding due to steam generator chemistry.
09/17/84	0200	Began power ascension.
09/18/84	0900	The reactor obtained 100% power.
09/28/84	2000	Began a manual shutdown for cycle 2 refueling/modification outage.
	2240	The reactor entered mode 2.
	2245	The turbine was tripped.
	2250	The reactor entered mode 3.
09/29/84	1923	The reactor entered mode 4 and was maintaining 340°F for a steam generator soak.
09/30/84	2359	The reactor was in mode 4. The refueling/modification outage continues.

PORV'S and Safety Valves Summary

No PORV's or safety valves were challenged during the month.

The following Licensee Event Reports (LER's) were sent during September 1984, to the Nuclear Regulatory Commission.

DESCRIPTION OF EVENT

1-84050

LER

A spurious signal from the chlorine detector caused a control room isolation (CRI) to occur at 0728C on August 7, 1984. Both units were in mode 1 at 100% power. The monitor was found functioning properly after performing Surveillance Instruction (SI)-240, "Functional Test of Control Room Air Intake Chlorine Detection System." The cause of the spurious signal could not be found nor could the signal be reproduced. The detector was determined to be operational and it was returned to service.

1-84051 Inspections at Sequoyah Nuclear Plant have identified the following additional items of noncompliance with Appendix R of 10 CFR 50. These inspections are part of an ongoing project to ensure compliance with Appendix R.

- 1. Interactions exist between train 'A' and 'B' at elevation 734 in the auxiliary control room. Train 'A' consists of cables for steam generator level control for motor driven auxiliary feedwater pump 1A-A (cables 1V1083A, 1V1061A, and 1V1063A). Train 'B' consists of cables for steam generator level control for motor driven auxiliary feedwater pump 1B-B (cables 1V1123B and 1V1103B). Also, at the location (cable tray PA-B) the above cables interact with those for auxiliary feedwater turbine driven pump (cables 1M811A and 1SG480).
- 2. Interactions exist between train 'A' and train 'B' at elevation 714 in the auxiliary building at column lines A-8 and Q. Train 'A' consists of cables to handswitch in the main control room (MCR) to AC breaker 1912 to 6900 shutdown boards (panel 6). Train 'B' consists of cables to handswitch in MCR of AC breakers 1728 and 1914 to 6900 shutdown boards 1B-B (panels 16 and 6). Also, interaction exists at same location with cable 1PP460B for handswitch in MCR to D/6 parallel operator or breaker 1726 and 1728 to 6900 shutdown board 1B-B (panel 6).
- 3. Interactions exist between train 'A' and train 'B' at elevation 714 at column lines A-8 and Q. Train 'A' consists of cables 1PP655A and 1PP653A to motor driven auxiliary feedwater pump 1A-A. Train 'B' consists of cables 1PP667B and 1PP665B to motor driven auxiliary feedwater pump 1B-B. Also, interaction exists at the same location for turbine driven auxiliary feedwater pump (cables 1SG240A, 1M1450, 1V3241A, 1V3251A, 1SG480, 1SG481, and 1SG461).

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(Continued)

DESCRIPTION OF EVENT

1-84051

LER

(Continued)

The action statement for Technical Specification 3.7.12 was satisfied by utilizing firewatches in the effected areas that were established by other Appendix R commitments. This action included the establishment of a roving fire watch in areas with fire detection and a dedicated fire watch in areas without fire detection.

1-84052

SI-144.2, "Control Room Emergency Ventilation System Test," was completed on both trains of the control room emergency ventilation system at 0100C on 08/17/84. Field calculations performed during the testing indicated both trains had passed the flow rate acceptance criteria of 4000 CFM ± 10%. Subsequent engineering review of the test discovered that the field calculation of the duct area was in error. The duct area was incorrectly calculated in the field to be 3.67 ft.² when the actual are 1 was 3.33 ft². Recalculation of the flow rates using the correct duct area determined that the flow rates had actually been lower than acceptable. The recalculated flow rates were 3580 CFM for train 'A' and 3290 CFM for train 'B'. Both trains of the control room emergency ventilation system were declared inoperable at 0300C on 08/17/84 requiring entry into LCO 3.0.3.

The control damper was adjusted to obtain an acceptable flow rate for train 'A'. Then train 'B' was checked and the flow rate found acceptable without further adjustment of the common damper. Both trains were declared operable at 0430C on 08/17/84.

1-84054

An 2033 CST on 8/27/84, at 100% power, SQNP Unit 1 experienced a reactor trip and turbine trip initiated from the steam flow-feed flow mismatch coincident with low steam generator level. Operations personnel complied with AOI-1, "Reactor Trip", and stabilized the unit in mode 3, hot standby, immediately after the trip. Simultaneous with the trip, reactor coolant system pressure channel PI-68-66 failed low. Prior to the trip all vital plant parameters were normal with no indication of steam flow-feed flow deviations or abnormal steam generator levels. Preliminary investigation revealed that the trip was actuated when instrumentation relay rack 1-R-15 experienced a loss of power. The relay rack breaker was reset and the trip cleared.

(Continued)

DESCRIPTION OF EVENT

1-84055

LER

This LER involves two separate incidents that occurred on 08/20/84. Two doors (A207 and A206) are located in the auxiliary building and give access to the roof. The outer door is a vital area boundary door and is secured locked. At approximately 0700 CST on 8/20/84, maintenance personnel notified security that work was required on the roof and that a security guard was posted at the door. Neither the maintenance personnel nor security personnel were aware that the outer door was a part of the Auxiliary Building Secondary Containment Enclosure (ABSCE) boundary. The work being performed on the roof required 110 volt power, so an extension cord was run through the door. At 0830 CST a Field Services supervisor (who was aware of the boundary) told the men to shut the door and notified Operations in the control room. An ASE was sent to the doors and verified that the doors were closed. The maintenance and security personnel still did not understand these requirements for closing the doors and they were reopened. The requirements for these doors as part of the ABSCE were reemphasized to maintenance management and the doors closed at 1330 CST. The second door-breaching can be attributed to failure of communication between first-line management and personnel performing the work.

The major contributing factor to these breaches was the failure of a plant procedure to list which doors are part of the ABSCE boundary. The control room does not issue security door control paperwork and was unaware of these breaches. The majority of auxiliary building doors are fire barriers and require a permit from Operations. The two doors involved are not fire doors, which also contributed to these two breaches of ABSCE.

1-84056

With unit 1 in mode 1 at 30% power and unit 2 in mode 1 at 10% power, instrument mechanics were working on the iodine channel to unit 1 lower compartment radiation monitor (1-RM-90-106C). The train 'A' power to panel 0-M-12 was lost due to its power breaker on 120 VAC vital board 1-II opening. The loss of train 'A' power to this panel caused an inadvertent containment ventilation isolation from monitor 1-RM-90-106 and an auxiliary building ventilation isolation from the spent fuel pool radiation monitor (0-RM-90-10?) which is powered from this same source. All safety systems actuated as designed when the signal was present and no abnormal actions occurred. Immediate communications between the instrument mechanics working on the monitor and Operations personnel allowed for quick identification of the breaker trip and action was taken to restore power to the panel and to rejet the CVI and ABI features.

(Continued)

DESCRIPTION OF EVENT

2-84013

LER

Prior to the event, on 08/11/84, with unit 2 at 100% power (2235 psi and 578°F) one of the pressurizer safety valves (2-SRV-68-653) experienced some slight leakage across the seat. Relief from this safety valve is designed to flow to the PRT, which has two rupture discs, if the tank pressure exceeds 85 psig. During operation prior to the event, PRT and safety valve parameters were montiored by operations personnel to ensure safe operation of the plant. All parameters including level, temperature, and pressure in the PRT were being maintained within prescribed operation limits. On the date of the event, 08/20/84, with the PRT parameters within limits, one of the two PRT discs ruptured prematurely. The rupture resulted in a small increase in radiation levels in the lower containment causing a containment ventilation isolation initiated by radiation monitor RM-90-106. There was no purging activity at the time of the event, therefore, the only isolation valves that closed were those to the radiation monitor itself and no inadvertent releases were made. Operations personnel initiated shutdown of the unit and notified the NRC by phone of the event in accordance with SNP REP IP-2. Unit shutdown (mode 3-Hot Standby) was completed at 1556 CST on 08/20/84.

2-84014

During a normal startup on 08/30/84, unit 2 was in mode 1 (2235 psig, 558°F) at 25 percent reactor power with the 'A' main feedwater pump in operation. The balance of plant operator was controlling steam generator levels with the main feedwater regulator valves in manual control. During switchover of the steam generator level controls from manual to automatic, steam generator #1 was overfed. The #1 steam generator high-high level trip setpoint was exceeded resulting in a turbine trip and feedwater isolation. The reactor operator immediately (with increasing level in the #1 steam generator) reduced reactor power in attempt to prevent an anticipated automatic reactor trip. With reactor power at approximately 5 percent the steam generator level shrink due to the feedwater isolation resulted in a lo-lo level in the #3 steam generator which tripped the reactor. The unit stabilized at 547°F following the reactor trip.

Diesel Generator Failure Reports

There were no diesel generator fail. reports transmitted during the month.

Special Reports

There were no special reports transmitted during the month.

Offsite Dose Calculation Manual Changes

There were no changes to the Sequoyah Nuclear Plant ODCM this month.

DOCKET NO. 50-327 DATE OCTOBER 6 1984 COMPLETED BY M. G. EDDINGS TELEPHONE (615) 870-6248

OPERATING STATUS

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1.	UNIT NAME: SEQUOYAH NUCLEAR PLANT, UNIT 1	NOTES:
2.	REPORT PERIOD: SEPTEMBER 1984	
з.	LICENSED THERMAL POWER(MWT): 3411.0	
4.	NAMEPLATE RATING (GROSS MWE): 1220.6	
5.	DESIGN ELECTRICAL RATING (NET MWE): 1148.0	
6.	MAXIMUM DEPENDABLE CAPACITY (GROSS MWE): 1183.0	
7.	MAXIMUM DEPENDABLE CAPACITY (NET MWE): 1148.0	
8.	IF CHANGES OCCUR IN CAPACITY RATINGS(ITEMS NUMBERS	
	3 THROUGH 7)SINCE LAST REPORT, GIVE REASONS:	
9.	POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWE):	
10.	REASONS FOR RESTRICTIONS, IF ANY:	

		THIS MONTH	YRTO-DATE	CUMULATIVE
11.	HOURS IN REPORTING PERIOD	720.00	6575.00	28512.00
12.	NUMBER OF HOURS REACTOR WAS CRITICAL	602.50	3997.10	18438.66
13.	REACTOR RESERVE SHUTDOWN HOURS	0.00	0.00	0.00
14.	HOURS GENERATOR ON-LINE	563.75	3786.80	17899.95
15.	UNIT RESERVE SHUTDOWN HOURS	0.00	0.00	0.00
16.	GROSS THERMAL ENERGY GENERATED (MWH)	1767263.66	11704071.84	57195922.14
17.	GROSS ELECTRICAL ENERGY GEN. (MWH)	573870.00	3819860.00	19198996.00
18.	NET ELECTRICAL ENERGY GENERATED (MWH)	549030.00	3657409.00	18434337.00
19.	UNIT SERVICE FACTOR	78.30	57.59	62.78
20.	UNIT AVAILABILITY FACTOR	78.30	57.59	62.78
21.	UNIT CAPACITY FACTOR (USING MDC NET)	66.42	48.45	56.32
22.	UNIT CAPACITY FACTOR (USING DER NET)	66.42	48.45	56.32
23.	UNIT FORCED OUTAGE RATE	21.70	27.37	20.14
24.	SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS	(TYPE, DATE,	AND DURATION	OF EACH):
				-

25. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:

NOTE THAT THE THE YR. -TO-DATE AND CUMULATIVE VALUES HAVE BEEN UPDATED.

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO.	50-327				
UNIT	Sequoyah One				
DATE	October 10, 1984				
COMPLETED BY	M. G. Eddings				
TELEPHONE	(615) 870-6248				

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MONTH	September		
DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	1058	17 .	1068
2	1069	18	1076
3	1071	19	1051
4	1070	20	1078
5	1067	21	9,69
6	1075	22	0 [~]
7	1074	23	0
8	1077	24	0
9	1071	25	0
10	1074	26	0
11	1074	27	0
12	1074	28	121
13	1076	29	257
14	1073	30	306
15	1073	31	
16	1076		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

(9/77)

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO.	50-327	
UNIT NAME	Sequoyah One	
DATE	October 5, 1984	
OMPLETED BY	M. C. Eddings	
TELEPHONE	(615) 870-6248	

REPORT MONTH SEPTEMBER

No.	Date	Type1	Duration (Hours)	Reason ²	Method Of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵		Cause & Corrective Action to Prevent Recurrence
	840921	F	156.25	A	1				Steam lea	ks on feedwater drain lines repaired
1 F: For S: Sch	ced meduled	B-Ma C-Re D-Re E-Op F-Ad G-Op	uipment intenanc fueling gulatory	e or 1 Restrirainin tive 1 Erro	lest riction ng & Lice	ense Examinatio	1 2 3 4 n 5	ethod: -Manual -Manual S -Automati -Cont. of Outage -Reduction -Other	c Scram. Existing	4 Exhibit G-Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG- 0161) 5 Exhibit I-Same Source

OPERATING DATA REPORT

DOCKET NO. 50-328 DATE OCTOBER 5,1984 COMPLETED BY D.C.DUPREE TELEPHONE (615)870-6248

OPERATING STATUS

1.	UNIT NAME: SEQUOYAH NUCLEAR PLANT, UNIT 2	NOTES:
2.	REPORT PERIOD: SEPTEMBER 1-30,1984	
з.	LICENSED THERMAL POWER(MWT): 3411.0	
4.	NAMEPLATE RATING (GROSS MWE): 1220.6	
5.	DESIGN ELECTRICAL RATING (NET MWE): 1148.0	
6.	MAXIMUM DEPENDABLE CAPACITY (GROSS MWE): 1183.0	
7.	MAXIMUM DEPENDABLE CAPACITY (NET MWE): 1148.0	
8.	IF CHANGES OCCUR IN CAPACITY RATINGS(ITEMS NUMBERS	
	3 THROUGH 7) SINCE LAST REPORT, GIVE REASONS:	
9.	POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWE):	
10.	REASONS FOR RESTRICTIONS, IF ANY:	

	THIS MONTH	YRTO-DATE	CUMULATIVE
HOURS IN REPORTING PERIOD	720.00	6575.00	20472.00
NUMBER OF HOURS REACTOR WAS CRITICAL	575.05	6124.75	16485.82
REACTOR RESERVE SHUTDOWN HOURS	0.00	0.00	0.00
HOURS GENERATOR ON-LINE	532.37	5987.99	16142.31
UNIT RESERVE SHUTDOWN HOURS	0.00	0.00	0.00
GROSS THERMAL ENERGY GENERATED (MWH)	1471237.65	19449577.30	51867645.11
GROSS ELECTRICAL ENERGY GEN. (MWH)	482390.00	6620740.00	17652680.00
NET ELECTRICAL ENERGY GENERATED (MWH)	459346.00	6373689.00	16991426.60
UNIT SERVICE FACTOR	73.94	91.07	78.85
UNIT AVAILABILITY FACTOR	73.94	91.07	78.85
UNIT CAPACITY FACTOR(USING MDC NET)	55.57	84.44	72.30
UNIT CAPACITY FACTOR (USING DER NET)	55.57	84.44	72.30
UNIT FORCED OUTAGE RATE	20.63	7.42	8.59
SHUTDOWNS SCHEDULED OVER NEXT & MONTHS	(TYPE, DATE,	AND DURATION	OF EACH):
			ne ann the instant set and
men		er fild vere vere men den som vere men mer met som vere me	an other most of the state state.
	NUMBER OF HOURS REACTOR WAS CRITICAL REACTOR RESERVE SHUTDOWN HOURS HOURS GENERATOR ON-LINE UNIT RESERVE SHUTDOWN HOURS GROSS THERMAL ENERGY GENERATED (MWH) GROSS ELECTRICAL ENERGY GENERATED (MWH) NET ELECTRICAL ENERGY GENERATED (MWH) UNIT SERVICE FACTOR UNIT SERVICE FACTOR UNIT AVAILABILITY FACTOR UNIT CAPACITY FACTOR(USING MDC NET) UNIT CAPACITY FACTOR(USING DER NET) UNIT FORCED OUTAGE RATE	HOURS IN REPORTING PERIOD720.00NUMBER OF HOURS REACTOR WAS CRITICAL575.05REACTOR RESERVE SHUTDOWN HOURS0.00HOURS GENERATOR ON-LINE532.37UNIT RESERVE SHUTDOWN HOURS0.00GROSS THERMAL ENERGY GENERATED (MWH)1471237.65GROSS ELECTRICAL ENERGY GENERATED (MWH)482390.00NET ELECTRICAL ENERGY GENERATED (MWH)459346.00UNIT SERVICE FACTOR73.94UNIT AVAILABILITY FACTOR (USING MDC NET)55.57UNIT CAPACITY FACTOR (USING DER NET)55.57UNIT FORCED OUTAGE RATE20.63	HOURS IN REPORTING PERIOD720.006575.00NUMBER OF HOURS REACTOR WAS CRITICAL575.056124.75REACTOR RESERVE SHUTDOWN HOURS0.000.00HOURS GENERATOR ON-LINE532.375987.99UNIT RESERVE SHUTDOWN HOURS0.000.00GROSS THERMAL ENERGY GENERATED (MWH)1471237.6519449577.30GROSS ELECTRICAL ENERGY GENERATED (MWH)482390.006620740.00NET ELECTRICAL ENERGY GENERATED (MWH)459346.006373689.00UNIT SERVICE FACTOR73.9491.07UNIT AVAILABILITY FACTOR73.9491.07UNIT CAPACITY FACTOR(USING MDC NET)55.5784.44UNIT CAPACITY FACTOR(USING DER NET)55.5784.44UNIT FORCED OUTAGE RATE20.637.42

25. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: November 27, 1984

NOTE THAT THE THE YR. -TO-DATE AND CUMULATIVE VALUES HAVE BEEN UPDATED.

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO.	50-328
UNIT	Sequoyah Two
DATE	October 5, 1984
COMPLETED BY	D. C. Dupree
TELEPHONE	(615)870-6543

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MC (TH	September		
DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	464	. 17 .	838
2	1088	18	1108
3	1090	19	1112
4	1092	20	1112
5	1012	21	: 1rh
6	193	22	1111.**
7	267	23	1113
8	843	24	1111
9	985	25	1109
10	N/A	26	1104
11	N/A	27	1099
12	N/A	28	970
13	N/A	29	N/A
14	253	30	N/A
15	262	31	
16	268		

INSTRUCTIONS

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On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

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UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO.	50-328
UNIT NAME	Sequoyah Two
DATE	October 5, 1984
COMPLETED BY	D. C. Dupree
TELEPHONE	(615)870-6248
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REPORT MONTH September

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method Of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	1.5	Cause & Corrective Action to Prevent Recurrence
8	840905	F	34.62	A	3				Low E.H.C.	pressure.
9	840909	F	19.93	A	3				Neutral transformer overvoltage.	
10	840910	F	3.98	G	3				Lo-Lo leve	el steam generator #4.
11	840910	F	3,30	A	3				Cond. DI was bypassed due to high Delta 'P' during start up causing the MFPT and S/G's to swing.	
12	840910	F	76.55	A	3					reg. valve to #4 S/G was sluggish lons during start-up. MFPT would not
1 F: Fo S: So (9/77)	heduled	B-Ma C-Re D-Re E-Op F-Adi G-Op	uipment intenanc fueling gulatory erator T ministra	e or Rest raini tive 1 Err	riction ng & Lic or (Expl	ense Examinat	ion	Method: 1-Manual 2-Manual S 3-Automati 4-Cont. of Outage 5-Reductio 9-Other	c Scram. Existing	4 Exhibit G-Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG- 0161) 5 Exhibit I-Same Source

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UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO.	50-328	12
UNIT NAME DATE	Sequoyah Two October 5, 1984	
COMPLETED BY	D. C. Dupree	
TELEPHONE	(615) 870-6248	

REPORT MONTH September

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method Of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵		Cause & Corrective Action to Prevent Recurrence
13	840928	S	49.25	с	1				Cycle 2 re	fueling/modification outage.
F: For S: Sct 9/77)	rced meduled	B-Ma C-Re D-Re E-Op F-Adr G-Op	uipment intenanc fueling gulatory erator T ministra	e or Rest raini tive 1 Err	riction ng & Lic or (Expl	ense Examinati	on 5	Method: -Manual 2-Manual S 3-Automati -Cont. of Outage -Reductio 2-Other	c Scram. Existing	4 Exhibit G-Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG- 0161) 5 Exhibit I-Same Source

The following significant maintenance items were completed during the month of September 1984:

Mechanical Maintenance

- 1. Intercept valve, 2-FCV-1-124, was leaking EHC fluid. A break in a weld was discovered and repaired.
- 2. The 1A stator cooling water pump was re-built and installed.
- 3. Pressurizer safety valve work continues.
- 4. Unstopped the boric acid line.
- 5. Unit 2 tripped due to insulation which fell out and shorted the buswork. The situation was corrected.
- 6. The EHC filters on the 2B pump were changed out following a loss of pressure due to clogged filters.
- 7. Due to a blown gasket in the B Auxiliary Control Air Compressor, it was rebuilt. Both the intake and discharge valves were replaced.
- 8. Performed SI-102, the monthly inspections on 2A-A, 1B-B, and 2B-B diesel generators.
- 9. Installed a Dressler-type coupling over a leak in a two-inch fire protection line at the diesel generator building. The coupling was installed to slow down the leak while a decision on the best method to isolate the leak was being made.
- 10. 1-RV-63-637 was believed to be leaking through. A pressure test was performed and the valve was good.
- 11. Installed Furmanite boxes on 1-VLV-3-549 and over an old box on 1-5-94. Both were injected with Furmanite to stop the leakage.
- Furmanite was injected into 1-FCV-1-98 on 1C1 MSR, 1-FCV-1-235 on 1C2, 3-549, 2-FCV-1-150, and into the B4 manway elbow on the B-1 heater, to stop leakage.
- The rekeying of some plant doors due to the loss of some important keys continues. These doors include CSSC and non-CSSC doors in all areas of the plant.
- 14. Replaced the boric acid conc. pump suction valve operator (1-FCV-62-164).
- 15. Scab plates were fitted up on ERCW lines because of leaks and thin places caused by cavitation downstream of the butterfly valve.
- 16. Shut-down to repair a leaking feedwater drain valve and a leaking steam generator #1 manway cover.

		ELECTRICAL MONTHLY REPOR	Plant Maintenance Summa	ry	Page 1 of 9
DATE	COMPONENT	FAILURE DESCRIPTION		CORRECTIVE ACTION	PRO#
09-07-84	2-RM-090-0120	PUMPS ON RAD MONITOR 120 AND 121 WILL NOT STAY ON EVEN WHEN FLOW THROUGH MONITOR 15 UP	BEARINGS ON MOTOR WORN OUT CAUSING ARMATURE TO SHORT OUT	REPLACED MOTOR AND CHECKED ROTATION. PERFORMED COUNT RATE ON MONITOR AND VERIFIED PROPER OPERATIONS RETURNED TO SERVICE	NONE
09-07-84	2-FSV-001-0182 -8	STEAM GENERATOR #2 BLOUDOWN HEADER FLOW ISOLATION VALVE STAYS OPEN WILL NOT CLOSE	LIMIT SWITCH OUT OF ADJUSTMENT	ADJUSTED LIMIT SWITCH	NONE
<u> </u>	2-FCV-001-0007 -8	STEAM GENERATOR #1 BLOWDOWN FLOW CONTROL VALVE STICKING IN CLOSED POSITION	LIMIT SWITCH BRACKET LOOSE	TIGHTENED LIMIT SWITCH BRACKET	2-84-144
09-07-84	1-BCTC-202-6.9 KVBRK	6.9KV BREAKER SHUTDOWN BOARD 18 WILL NOT OPERATE		SWITCHED OUT BREAKER AND REBUILT	NONE
	2-FCV-003-0047 -6		DIRTY CONTACTS ON INTERLOCKS CONTACTS WERE NOT DROPPING OUT	CLEANED CONTACTS ON INTERLOCKS	2-84-136
09-07-84	2-FCV-001-0029	INSULATION FOR LEADS IS DETERIORATED ON #4 MAIN STEAM ISOLATION VALVE	LEAGS DETERIORATED ALLOWING MOISTURE TO CORRODE LEAD COVERS AND COULD POSSIBLE SHORT OUT CONTROL POWER MSIV	CLEANED ELECTRICAL LEADS AND REPLACED COVERS	NONE
09-07-84	2-FCV-001-0029 -T	GREEN LIGHT WILL NOT BURN WHEN VALVE IS IN CLOSED POSITION		CLEAHED CONTACTS ON SNAP LOCK	NONE
09-07-84	2-HTRA-030-015 6-8	AUX. BLDG. GAS TREATMENT HUMIDIFIER HEATER WILL NOT COME ON	BAD RELAY ON HEATER	REPLACED RELAY	NGNE
09-07-84	0-НТСК-234-110 S	RED LIGHT DOES NOT COME ON GREEN LIGHT STAYS ON ALL TIME WHEN THEMOSTATE			NONE

		ELECTRICAL MONTHLY REPOR	Plant Maintenance Summa	ry	Page 2 of
DATE	COMPONENT	FAILURE DESCRIPTION		CORRECTIVE ACTION	PR0#
		IS TURNED UP ON CVCS B1 HEAT TRACE			
09-07-84	1-AC8-201	BREAKER ON 480 VOLT SHUTDOWN BOARD WILL NOT OPERATE	TRIP COIL AND CHARGING MOTOR BURNT UP	REPLACED CHARGING MOTOR AND TRIP COIL	NONE
09-10-84	2-FSV-067-0182 -8	WHEN VALVE IS FULLY OPEN ON SIS PUMP ROOM COOLER 28 STATUS LIGHT IN CONTROL ROOM WILL NOT LIGHT UP	LIMIT SWITCHES OUT OF ADJUSTMENT	ADJUSTED LIMIT SWITCH	NONE
09-10-84	0-TC-234-0311- 5	RESET ON TRIP POINT DRIFTS AROUND ON CVCS ELECTRIC HEAT TRACE CONTROL PANEL 82-2	BAD PLUG ON HEAT TRACE CONTROLLER	REPAIRED PLUG FOR CONTROLLER, CHECKED CONTACTOR AND HEAT TRACEFOR OPERATIONS	NONE
09-10-84 - 20-	0-CHR-031-0338 -8	BUBBLES IN SIGHT GLASS OF SHUT DOWN ROOM WATER CHILLER PACKAGE A	FREON LOW	ADDED FRECH TO PROPER LEVEL WITH R-22	NONE
09-10-84	1-DGRP-082-001 A-A	ALARM LIGHT WILL NOT FUNCTION PROPIRLY ON 1A-A DIESEL GENERATOR RELAY PANEL	NO CAUSE FOUND	FUNCTIONALLY CHECKED WORKING PROPERLY AT THIS TIME	NONE
09-11-84	2-FCV-067-0354	LIGHT IN CONTROL ROOM WILL NOT FOLLOW LIMIT SWITCH ON VALVE-PENETRATION ROOM COOLER A3 ERCW FLOW CONTROL VALVE	LIMIT SWITCH OUT OF ADJUSTMENT	ADJUSTED LIMIT SWITCH	NONE
09-11-84	2-FCV-067-0350	LIGHT IN CONTROL ROOM WILL NOT FOLLOW LIMIT SWITCH ON VALVE -PENETRATION ROOM COOLER A2 ERCW FLOW CONTROL VALVE	LIMIT SWITCH OUT OF ADJUSTMENT	ADJUSTED LIMIT SWITCH	NONE
09-11-84	0-MTRB-032-002	BREAKER WILL NOT TRIP OUT	TRIP COIL BURNT OUT ON	PLUGGED IN NEW COIL,	NONE

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		ELECTRICAL MONTHLY REPOR	Plant Maintenance Summa RT FOR SEPTEMBER	iry	Page 3 of 9
DATE	COMPONENT	FAILURE DESCRIPTION	CAUSE OF FAILURE	CORRECTIVE ACTION	PR0#
	5	ON CONTROL SERVICE AIR COMPRESSOR A	BREAKER	REPLACED MOTOR CUT OFF SUITCH ACTUATOR ADJUSTED MOTOR CUT OFF SWITCH FUNCTIONALLY TESTED WORKING PROPERLY AT THIS TIME	
09-11-84	2-XS-013-0161- A	PYROTRONICES DETECTOR IN UNIT #2 AUX BUILDING HEAT VENT ROOM ALARM STAYS IN FOR NO APPARENT REASON	BAD SMOKE DETECTOR	REPLACED SMOKE DETECTOR	NONE
09-11-84	1-FCV-072-0013	SEAL IN INTERLOCK ON CONTACTOR STUCK IN CLOSED POSITION ON CONTAINMENT SPRAY RECIRCULATING PUMP 18		CLEANED CONTACTS WITH FREON SOLVENT (MS-180) FUNCTIONALLY TESTED CONTACTS. NO FURTHER TESTS REQUIRED	NONE
09-11-84 -21 -	1-FCV-063-0073	SEAL IN INTERLOCK ON CLOSE CONTACTS IS STUCK IN THE CLOSED POSITION ON THE CONTAINMENT SUMP FLOW ISOLATION VALVE		CLEANED CONTACTS WITH FREON SOLVENT (NS-180) FROM APPROVED LIST OF SOLVENTS (EMSL18) FUNCTIONAL TESTED CONTACTS NO FURTHER TEST REQUIRED	NONE
09-11-84	1-0XF-201-001B -B	SI HEAT TRACE TRANSFORMER "B" MOTOR STARTER AUXILIARY CONTACTS RED AND BLACK PUSH BUTTONS. BLACK BUTTONS SEEM TO BE HIGHER THAN NORMAL	CONTACTS DIRTY	CLEANED CONTACTS OF A GUMMY STICKY SUBSTANCE	NONE
09-12-84	2-LCV-003-0175 -A	VALVE WAS VERIFIED CLOSED LOCALLY EUT RED AND GREEN LIMITS ARE ON	ACTUATION ARM ON LIMIT SWITCH OF AUX > FEEDWATER LEVEL CONTROL VALVE OUT OF ADJUSTMENT	ADJUSTED ACTUATOR ARM	2-84-148
09-14-84	1-BCTB-201-DJ/ 9D-A	C&S AIR COMPRESSER A BREAKER ON 480VOLT SHUT DOWN BOARD 1A2-A WOULD NOT WORK	CONTACTS STUCK AND CORRODED, SETTINGS OUT OF LIMITS	INSTALLED SPARE BREAKER	NONE

		ELECTRICAL MONTHLY REPOR	Plant Maintenance Summa	iry	Page 4 of 9
DATE		FAILURE DESCRIPTION		CORRECTIVE ACTION	PRO#
09-14-84	0-LOCL-013-061 6	ALARM ON PANEL WILL NOT CLEAR	GREEN CONDUIT SHORTED OUT IN THE EXTERNAL ALARM HORNS CABLE EE5668 IS NON CONFORMING WITH CONTROL DRAWING PRINT #45W1699-33	COMPLETED TO USE ORANGE	NONE
09-17-84	0-ACU-311	AA CONTROL ROOM AIR CONDITIONING UNIT WILL NOT STAY RUNNING		ADJUSTED SWITCH AND VALUE ON WATER FLOW	NONE
09-18-84	1-GEN8-082-000 1A-A	DIESEL FAILED TO LOAD UP TO 4 MEGAWATTS ELECTRICAL IN 60 SECONOS PER SURVAILANCE INSTRUCTIONS 7 PROBLEM CLEARED WHEN OPERATOR WENT TO MANUAL		REPLACED LOAD SENSOR MODULE	1-84-287
09-18-84 - 22-	0-LOCL-013-061 2	ZONE 308 RELAY IS ENGERGIZED AND TROUBLE ALARM IS PRESENT ON PANEL 0-L-612	NO FAULT COULD BE FOUND	FUNCTIONALLY TESTED AND WORKING PROPERLY AT THIS TIME	HONE
09-18-84	1-XA-063-0064	WHEN STARTING 1A RHR PUMP FOR SI128 WINDOW #20 CAME IN AND WOULD NOT CLEAR	ARM ON VALVE STEM DOESN'T HIT SWITCH LEVER CAUSING LIMIT SWITCH NOT TO OPEN	INSTALLED OFF SET LIMIT ARM FOR OPERATION	NONE
09-18-84	1-RLY-082-00P0	RELAY IS LOOSE TO THE TOUCH CHATTERS WHEN TOUCHED	LOOSE IN PLUG	REPLACED PO RELAY AND CHECKED CPERATION OF SAME. DIESEL GENERATOR NOT INOP AT ANY TIME	NONE
09-20-84	0-FCV-032-0094	VALVE OCCUSIONALY OVER TRAVELS DEFEATING AIR PURGE STEP FOR DRYER #2	LIMIT SWITCH ARM OUT OF ADJUSTMENT	ADJUSTED ARM ON LIMIT SWITCH	NONE
09-20-84	2-FCV-063-0084	SIS CHECK VALVE ISOLATION HEADER LEAK TEST ISOLATION VALVE IS IN CLOSED POSITION VALVE INDICATING BOTH RED AND GREEN	LIMIT SWITCH ACTUATOR SLIPPED ON SHAFT CAUSING VALVE INDICATING LIGHTS RED AND GREEN TO STAY ON	ADJUSTED LIMIT SWITCH ACTUATOR ON VALVE	NONE

		ELECTRICAL MONTHLY REPOR	Plant Maintenance Summa	ry	Page 5 of 9
DATE	COMPONENT	FAILURE DESCRIPTION	the second s	CORRECTIVE ACTION	PRO#
09-21-84	1-TE-072-0011- 8	CONTA NMENT SPRAY PUMP B MOTOR INBOARD BEARING TEMPERTURE ELEMENT NOT WORKING	EMPLOYEES WERE USING THIS CONDUIT FOR A STEP ON CONTAINMENT SPRAY PUMP B MOTOR. CONDUIT IS BROKEN AND TEMPERTURE ELEMENT IS COMPLETELY REMOVED FROM TEMPERTURE WELL	REPLACED CONDUIT AND INSTALLED TEMPERTURE ELEMENT TJ TEMPERTURE WELL	NONE
09-24-84	1-LCV-062-0192 -A	A BORIC ACID EVAPORATOR DISCHARG VALVE WILL NO FULLY CLOSE	LIMIT SWITCH OUT OF ADJUSTMENT WILL NOT LET VALVE FULLY CLOSE	ADJUSTED LIMIT SWITCH	NONE
09-25-84	0-RE-090-0101- A			MECHANICAL REPLACED THE PUMP AND ELECTRICAL REPLACED THE MOTOR	NONE
09-25-84	2-LCV-003-0174 -8	PREVENTIVE MAINTANCE	NO FAILURE FUNCTIONALLY CHECK 74-8 TIMER TO ENSURE IT WILL TIME OUT (30 SECOND TIME DELAY)	CHECKED TIMER AND FOUND TO BE WORKING. SET TIMER FROM 41 SEC. TO 30 SEC. STOP WATCH 902147 1-11-85 TESTED PER MI6.20	2-84-142
09-25-84 - 23-	0-XA-013	NO FAILURE	MODIFICATION WORK UNDER WP 10009P ECN 5237	LIFT ZONE WIRE AT PANEL FOR ZONES 349%350 USING MI6.20 PERFORM SI 234.4 AFTER MODIFICATION INSTALL JUMPER FOR ZONES 349%350 RECORD PER MI6.20 TO CLEAR TROUBLE LIGHT CN PANEL, REMOVE JUMPER AFTER MODIFICATION	NONE
09-25-84	0-HTCK-00905&P		THIS CIRCUIT ONLY HAS A PRIMARY AND THE TEMPTURE IS 165		NONE
09-25-84		HEAT TRACE REPORTED BAD ON MRA248393		AMPS WERE CHECKED AND FOUND TO BE CORRECT	NONE

		ELECTRICAL MONTHLY REPOR	Plant Maintenance Summa	ry	Page 6
DATE	COMPONENT		CAUSE OF FAILURE	CORRECTIVE ACTION	PRO#
09-25-84	0-HTCK-234-010 0P&S	SAMPLE VALVE 0-VLV-062-1056 WAS REPORTED BAD ON MRA248393	HEAT TRACE CIRCUIT TO BORIC ACID TANKS "C" WAS REPORTED BAD ON MRAE248393	AMPS WERE CHECKED AND HEAT TRACE WORKING PROPERLY AT THIS TIME	NONE
09-26-84	2-MTRB-062-023 2-8	PUMP KEEPS TRIPPING OUT ON BORIC ACID TRANSFER PUMP 28-B	LEAD TO MOTOR STARTER HAD BAD CRIMP ON TERMINAL LUG	REPLACED BAD CRIMP ON TERMINAL LUG TO MOTOR STARTER PER MODIFICATION AND ADDITIONS INSTRUCTION 12	NONE
09-26-84	1-MTRB-077-000 6	REACTOR COOLANT DRAIN TANK PUMP 18 KEEPS TRIPPING OUT	THERMOAL OVERLOADS BAD	REPLACED OVERLOADS IN BREAKER	NGNE
09-27-84	1-AHU-313-0001 8-8	TEMPERTURE IN ROOM TOO WARM 89	480 VOLT BOARD ROOM 18-8 AIR HANDLING UNIT AND CHILLER PACKAGE WILL NOT REMAIN RUNNING	CHECKED UNIT AND IT IS OPERATING PROPERLY AT THIS TIME	NONE
09-27-84	0-CHR-313-0338 -A	NO FAILURE OIL LOW	OIL BELIEVED TO BE LOW	CHECKED OIL LEVEL AND FOUND TO BE NORMAL (NO OIL ADDED)	NONE
09-27-84 -24- 1	0-FCO-30-0122	CASK LOADING AREA DAMPER DID NOT ISOLATE ON AUXILIARY BUILDING ISOLATION SIGNAL FROM O-RM-90-0102 FUEL POOL RAD MONITOR	DIRTY CONTACTS	CLEANED CONTACTS CHECKED RELAY WORKING PROPERLY AT THIS TIME	NONE
09-27-84	1-AHU-030-0162	FUEL POOL RAD MONITOR O-RM-90-102 DID NOT STOP ON AUXILIARY BUILDING ISOLATION SIGNAL	DIRTY CONTACTS	CLEANED CONTACTS CHECKED RELAY WORKING PROPERLY AT THIS TIME	NONE
09-27-84	1-AHU-030-0159	1A AUXILIARY BUILDING GENERATOR EXHAUST FAN DID NOT STOP ON ISOLATION SIGNAL FROM 0-RM-90-102 FUEL POOL MONITOR	DIRTY CONTACTS	CLEANED CONTACTS CHECKED RELAY WORKING PROPERLY AT THIS TIME	NONE

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DATE	COMPONENT	FAILURE DESCRIPTION	the second se	CORRECTIVE ACTION	PR0#
09-28-84	1-INVB-250-000 N-E	RESULTS OF TESTS RAN AT WATTS BAR IN THE WEEK OF JUNE 19 1984 ON VITAL INVERTERS	REPLACE C1 AND C6 CAPACTOR BANKS PER MEMO FROM JM MCGRIFF TO JB KRELL DATED JULY 25 1984 MEMO ATTACHED	REPLACED CAPACITOR BANKS C1 AND C6 PER MODIFICATION AND ADDITIONS INSTRUCTION 9 & 12	NONE
09-28-84	1-INVB-250-000 L-D	RESULTS OF TESTS RAN AT WATTS BAR IN THE WEEK OF JUNE 19 1984 ON VITAL INVERTER	REPLACED C1 AND C6 CAPACITOR BANKS PER MEMO FROM JM MCGRIFF TO JB KRELL DATED JULY 25 1984 MEMO ATTACHED	REPLACED CAPACITOR BANKS C1 AND C6 PER MODIFICATION AND ADDITIONS INSTRUCTION 9 & 12	NONE
09-28-84	1-INVB-250-000 R-F	RESULTS OF TESTS RAN AT WATTS BAR IN THE WEEK OF JUNE 19 1984 ON VITAL INVERTER	REPLACED C1 AMO C6 CAPACITOR BANN 3 PER MEMO FROM JM MCGRIFF TO JB KRELL DATED JULY 25 1984 MEMO ATTACHED	REPLACED CAPACITOR BANKS C1 AND C6 PER MODIFICATION AND ADDITIONS INSTRUCTION 9 & 12	NONE
09-28-84 1 25 1	1-INVB-250-000 T-G	RESULTS OF TESTS RAN AT WATTS BAR IN THEWEEK OF JUNE 19 1984 ON VITAL INVERTER	REPLACED C1 AND C6 CAPACITOR BANKS PER MEMO FROM JM MCGRIFF TO JB KRELL DATED JULY 25 1984 MEMO ATTACHED	REPLACED CAPACITOR BANKS C1 AND C6 PER MODIFICATION AND ADDITIONS INSTRUCTION 9 & 12	NONE
09- 29- 84	2-GENE-082-000 24-A	FUSES ON EXITATION PANEL OF UNIT 2 DIESEL GENERATOR 2A-A KEEPS BLOWING AFTER FUSES HAVE BEEN REPLACED ON FU-3 AND FU-4	24 VGLT D.C. DRIVE MOTOR BAD	REPLACED 24 VOLT D.C. DRIVE MOTOR AND POJUSTED CAM LIMITS	2-84-163
09-29-84	1-80E-250-00NG -F	FREQUENCY METER WILL NOT CALIBRATE ON THE 120 VOLT A.C. VITAL INSTRUMENT POWER 1-III		REPLACED METER	NONE
09-29-84	1-FCV-001-0004	MAIN STEAM ISOLATION VALVE LOOP 1 LIMIT SWITCH TEST LIGHT STAYS ON	LIMIT SWITCH B(4)	INSTALLED LIMIT SWITCH AND VERIFIED WORKING PROPERLY AT THIS TIME	NONE
09-29-84	1-FCV-001-0022	VALVE IS CLOSED AND GREEN LIGHT WILL NOT ILLUMINATE	LIMIT SWITCH BAD	REPLACED LIMIT SWITCH	NONE

		ELECTRICAL MONTHLY REPO	Plant Maintenance Summa	iry	Page 8 of
DATE	COMPONENT		CAUSE OF FAILURE	CORRECTIVE ACTION	PRO#
09-29-84	1-FCV-030-0056	LIMIT SWITCH OUT OF ADJUSTMENT	DIRTY CONTACTS	CLEANED CONTACTS	NONE
09-29-84	1-ZS-087-0023	DURING MODIFICATION FOR THE FIFTH VITAL BATTERY THE POWER CABLE WAS DAMAGED	CONSTRUCTION DRILLED INTO POWER CABLE TO FCV-87-23 DAMAGEING THE CABLE	TEMPERARY CABLE WAS CONNECTED BETWEEN THE JUNCTION BOXES FOR VALVES FCV-87-23 AND THE POWER CABLE FROM THE MAIN CONTROL ROOM TO THE JUNCTION BOX	1-84-335
09-29-84	1-MTRB-030-010 3	FAN DID NOT STOP ON ISOLATION SIGNAL FROM O-RM-90 FUEL POOL RAD MONITOR IN AUXILIARY BUILDING	ND PROBLEM FOUND	FUNCTIONALLY YESTED OPERATING NORMAL AT THIS TIME	NONE
09-29-84 -26-	2-PS-003-0160A -B	LOOP 1 PIPE BREAK LIGHT COMES ON WHEN TERRY TURBINE STARTS	CABLE 2V1045(WIRE #15F3 AND #15°H-120V.D.C) LAID DOWN WITH CABLE 25G501 (WIRE #8FP11 AND M3D24-24VAC) IN PS-3-1608	CHANGED TO CORRESPOND WITH DRAWING 450603-5	2-84-142
09-29-84	2-FSV-063-0077 -A	LIMIT SWITCH WILL NOT CLEAR	ACTIVATOR ARM ON LIMIT SWITCH OUT OF ADJUSTMENT	ADJUSTED ACTIVATOR ARM ON LIMIT SWITCH	NONE
09-29-84	1-FCV-087-0021	ND FAILURE RETURNETO NORMAL CONFIGURATION 1-FCV-87-21 PER SMI-1-87-6	NO FAILURE	RETURNED TO NORMAL CONFIGURATION	NONE
09-29-84	2-ZS-003-0048	STEAM GENERATOR #2 FEED WATER REGULATOR VALVE OPEN LIMIT SWITCH IS NOT MADE UP	ND PROBLEM FOUND	HAD OPERATION STROKE VALVE SEVERAL TIMES OPERATING NORMAL AT THIS TIME	NONE
09-29-84	2-AHU-313-0514 -B	480 VOLT BOARD ROOM AHU 28 WILL NOT RUN	THERMOSTAT AIR LINE CRIMPED BLOCKING UNIT FROM RUNNING	UNCRIMPED AIR LINE	NONE
09-29-84	1-FCV-001-0147	MAIN STEAM ISOLATION	LEVER ON LIMIT SWITCH OUT	ADJUSTED LEVER ON LIMIT	NONE

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		ELECTRICAL MONTHLY REPOR	Plant Maintenance Summa	ary	Page 9 of 9
DATE	COMPONENT		CAUSE OF FAILURE	CORRECTIVE ACTION	PR0#
		VALVE BY PASSINDICATES THROTTLED WHEN VALVE IS OPEN	OF ADJUSTMENT	SWITCH	
09-29-84	1-FCV-001-0022	MAIN STEAM ISOLATION VALVE (LOOP 3) LIMIT SWITCH INDICATES OPEN WHEN VALVE IS CLOSED	LIMIT SWITCH OUT OF ADJUSTMENT	ADJUSTED LIMIT SWITCH	NONE
84-08-09	0-MTRB-032-002 5	TRIP COIL SHORTED OUT AND BURNT UP	SHORT TO GROUND	REPLACED TRIP COIL	NONE

65 records listed.

(Continued)

Instrument Maintenance

1. During the month, there were eight auxiliary building vent isolations and one containment vent isolation on unit 1. Seven of the eight ABIs were the result of high background radiation levels primarily associated with spent fuel pit water quality. The other ABI occurred during filter change out on the auxiliary building radiation monitor. The unit 1 CVI was caused by flow switch chatter due to low sample flow in the lower containment radiation monitor.

During the month, a modification to the upper level discriminator circuit, ECN-6151, was incorporated on total gas and air particulate channels. Also time delay relays were installed, ECN6137, for the spent fuel pit area monitors.

- 2. The monthly calibration of the UHI level switches found all switches for unit 1 and 2 within Technical Specification tolerance.
- 3. Temperature and flow instrument channel for the flow of the diffuser pond to the river received maintenance this month. Lightning storms during the month caused several failures. Flow element 230B was replaced and both temperature channels were repaired. All channels were recalibrated and remain in service. Lightning arrestors have been installed and proper grounding of the channels was implemented.

Unit 1

- 1. Containment sump level transmitter 1 LT-63-178 drifted high. The filled senseline was topped off with oil, the transmitter recalibrated and returned to service.
- Implemented FCR 2634, ECN 6055, which restored a 0-600 psi channel for RCS pressure. This FCR fully implemented the requirement for RCS wide range pressure PAM channels. TACF 84-0083-68, which was initiated in July, was cleared.
- 3. During the month, number 2 governor valve experienced fluctuations which resulted in minor load changes. The fluctuations were originating in the speed error channels of the AEH controller. Defective electrolytic capacitors were found on three circuit boards. Six (6) circuit boards were replaced with spares during the forced outage of 9/24. The spare boards were calibrated and returned to service. No problems with the AEH controller were experienced during startup and subsequent operation.

(Continued)

Instrument Maintenance (Continued)

Unit 2

- 1. Following the reactor trip on 8/5/84, a pipe break detection alarm occurred for loop 1 of turbine driven auxiliary feedwater system. Both pipe break pressure switches were out of tolerance and a wiring error was found. The wiring was corrected. A special test instruction was written and performed to resolve discrepancies found during troubleshooting. During performance of the test a cable was found improperly labled. The pipe break switches on the other loops were inspected. A switch on loop 3 was found corroded and was replaced. All switches are scheduled to be replaced with 0588 qualified replacements during U2C2 outage.
- After a series of reactor trips early in the month, Operations Section requested that feedwater and steam generator controls be calibrated. Controllers and reg valves were checked for loops 2 and 3. Minor adjustments were made. No problems were encountered during the subsequent startup.
- 3. Other maintenance work is shown on the attached list.

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	INST	RUMENT	MA	INTENANCE	MONTHLY S	Summary	10-10-84			
MR.COM	PU	FUNC	SYS	ADDRESS.	DATE	DESCRIPTIO	DN	•••••		CORRECTIVE ACTION
A04781	8 () ps	082	262	09/17/84				PRESSURE SWITCH	THE PRESSURE SWITCH WAS NOT OPERATING PROPERLY. REPLACED THE DEFECTIVE SWITCH CALIERATED AND LEFT IN SERVICE
A10187	8 0	PI	032	88	09/26/84	0-PI-032-0	88, VERIFY/	RECAL AS N	ECESSARY	OUT OF CAL. RECAL AND RETURNED TO SERVICE.
A10194	4 2	PDS	065	970	09/19/84				IS INOP. FOUND INVESTIGATE AND	THE CONTACTS ON THE PRESSURE SWITCH WERE FOUND DIRTY. CLEANED THE CONTACTS VERIFIED CALIBRATION AND RETURNED TO SERVICE.
A10226	4 0	FR	027	2304	09/13/84	0-FR-027-2	230A, RECOR	DER IS DRIV	ien hard upscale	THE RECORDER WAS DRIVEN UPSCALE DUE TO THE FLOW MODIFIER. OUT OF CAL. REMOVED THE FLOW MODIFIER FROM SERVICE RECALIBRATED & RETURNED TO SERVICE
A10227	8 2		003	172	09/27/84			ce relay su h temperatu		A REQUEST WAS SUBMITTED TO CHANGE THE RELAY TO A HI TEMP RELAY ON THE STEAM GENERATOR #2 LEVEL CONTROL VALVE. REPLACED THE RELAY ON TEH POSITIONER WITH A HIGH TEMP. MODULE. VERIFIED OPERATION AND RETURNED TO SERVICE.
A10227 - 30-	92	LCV	003	173	09/27/84			CE RELAY SU H TEMP, MOD		A REQUEST WAS SUBMITTED TO CHANGE THE RELAY TO A HI TEMP. RELAY ON THE STEAM GENERATOR #2 LEVEL CONTROL VALVE. REPLACED THE RELAY ON THE POSITIONER WITH A HIGH TEMP. MODULE. VERIFIED OPERATION AND RETURNED TO SERVICE.
A12155	9 1	RT	090			OCCASSION	ALLY BRING	ING IN INST	DOWNSCALE MALF. ALARMS	THE MODULE WAS FAILING DOWN SCALE BRINGING IN THE INSTRUMENT MALFUNCTION ALARM. REPAIRED THE HIGH VOLTAGE CONNECTOR ON THE DETECT. &REPLACED POWER SUPPLY.
A12164	7 1	PIS	087	22	09/18/84	1-PIS-087- PRESSURE I	22, CHECK	PIS-87-22 0 IS TOO HIG	ALIBRATION. H.	DVM READOUT METER BAD. REPLACED THE DVM READOUT METER AND VERIFIED CALIBRATION
A12167	0 1	FI	063	920	09/01/84	GALS WHEN	THE FI-63	-92A IS IND	INDICATING 500 DICATING 0 GPM, ON; PLEASE REPAIR.	THE INDICATOR WAS INDICATING 500 GALLONS PER MINUTE WHEN A SIMULAR INDICATOR WAS READING 0. THE TOLLERANCE RANGE WAS CHECKED AND EVERYTHING WAS WITHIN ACCEPTANCE CRYTERIA.
A12174	1 1	FIS	063	43		OR REPAIR	AS NECESS		IN INDICATING NO	HAIRLINE INDICATOR HAD FALLEN OFF OF FIS. REINSTALLED THE HAIRLINE SCALE, ZERGED THE INDICATOR VERIFIED OPERATION AND RETURNED TO SERVICE.

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1	INST	RUMENT	MA	INTENANCE	MONTHLY	SUMMARY	10-1	0-84					2	
MR.COM	v u	FUNC	SYS	ADDRESS.	DATE	DESCRIPTIO	N	•••••	•••••					CORRECTIVE ACTION
A122654	4 1	ц	068	320	09/24/84	1-LI-068-3 LI-68-320.		ERFORM	M LOOP	CALI	BRATIC	on chi	eck on	THE SENSE LINES AND THE TUBING HAD LOST PARTIAL LEG BECAUSE OF A LEAK ON SENSE LINE GOING TO 1-PT-69-322. BACK FILLED, VERIFIED CAL AND RETURNED TO SERVICE.
A231904	4 1	PS	062	244	09/10/84	1-PS-062-2 BE REPLACE		SWITCH	H DOES	5 NOT	REPEAT	r. N	EEDS TO	THE PRESSURE SWITCH WAS NOT REPEATING. REPLACED THE DEFECTIVE SWITCH, CALIBRATED AND RETURNED TO SERVICE.
A233378	3 2	FCV	003		09/11/84	2-FCV-003-	, CHE	CK STR	ROKE O)F 2-F	CV-3-1	1418		THE FEEDWATER CONTROL VALVE WAS NOT OPERATING PROPERLY. CHECKED THE STROKE OF THE VALVE, AND RETURNED TO SERVICE
A233379	2	FCV	003	90	09/11/84	2-FCV-003-	90, C	HECK S	STROKE	E OF 2	E-FCV-3	3-90		THE FLOW CONTROL VALVE WAS NOT OPERATING PROPERLY. VERIFIED OPERATION OF VALVE STROKE AND RETURNED TO SERVICE
A233384	4 0	PS	032	88	09/25/84	0-PS-032-8	8, CH	ECK C	ALIB O	F P.S	i.			SWITCH OUT OF CAL. RECAL SWITCH.
A233385	5 0	PS	032	88A/8	09/25/84	0-PS-032-8	BA/B	, CHECK	K CALI	B OF	P.S.			PS-32-88 OUT OF CAL. RECAL SWITCH.
A233386	5 1	FCV	001	107	09/25/84	1-FCV-001-	107,	CHECK	STROK	EOF	STM DU	IMP VI	ALVE	CV STROKE OUT OF CAL. RESTROKED VALVE
A233388	3 1	LT	063	109	09/26/84	1-LT-063-1 SERVICE.	09,0	HECK C	CALIB.	OFL	T-RETU	IRN TI	D	THE AMPLIFIER BOARD WAS FOUND BAD DUE TO AN ELECTRONIC PROBLEM. REPLACED THE AMPLIFIER BOARD, RECALIBRATED XMTR AND RETURNED TO SERVICE.
A239986	5 1	PT	068	322	09/25/84		22, RI	EPLACE	E DEFE	CTIVE	FITTI	ING A	ND SENSE	THE TRANSMITTER HAD LOST PARTIAL LEG
-31.						LINE								BECAUSE OF A LEAK ON THE SENSE LINES. REPLACED THE FITTINGS AND SENSE LINES.
'A247597	2	PS	003	1408	09/19/84	2-PS-003-1 MR A292752		*I* CL	lear d	ISCRE	PANCIE	s foi	nio on	THE SWITCH WAS FOUND WITH CORROSION ON IT CAUSING IT NOT TO PERFORM ITS INTENDED FUNCTION. REPLACED THE SWITCH, CALIBRATED AND RETURNED TO SERVICE.
A279294	1 2	xx	003	LOOP2	09/13/84	2-XX-003-L	0092	CHECK	K LOOP	2 50	LVL C	ONTRO	DLLER	A REQUEST WAS MADE TO VERIFY LOOP OPERABILITY. VERIFIED OPERATION AND RETURNED TO SERVICE. NO PROBLEM FOUND.
A279295	5 2	xx	003		09/13/84	2-XX-003-,	CHECI	K LOOP	935G	LEVE	1. CONT	ROLLE	ERS	A REQUEST WAS MADE TO VERIFY LOOP OPERABILITY. VERIFIED OPERATION AND RETUNED TO SERVICE. NO PROBLEM WAS FOUND.
A279297	2	FCV	003	35	09/11/84	2-FCV-003-		HECK S	STROKE	OFS	g Loop	4 FI	ROM	THE FLOW CONTROL VALVE FOR THE STEAM GENERATOR #1 FEEDWATER WAS NOT OPERATING PROPERLY. VERIFIED PROPER VALVE STROKE OPERATION AND RETURNED TO SERVICE
A282207	1	PM	890	SPARE	09/12/84	1-PM-068-SI (FUNCTIONA				C			TORES.	AN OPERATIONAL CHECK WAS REQUESTED FOR THE PRESSURE MONITOR. PERFORMED FUNCTIONAL CHECK.

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	INS	TRUMENT	r MA	INTENANCE	MONTHLY S	SUMMARY 10-10-84	
MR.COM	PU	FUNC	SYS	ADDRESS.		DESCRIPTION	CORRECTIVE ACTION
A28221	0	LT	063	109	09/25/84	1-LT-063-109, CHECK CAL OF TRANSMITTER AFTER BLIND FLANGES ARE INSTALLED	A REQUEST WAS MADE TO VERIFY CALIBRATION. VERIFIED CALIBRATION AND FOUND NO PROBLEMS.
A28295	2 :	2 FCV	003	103	09/10/84	2-FCV-003-103, FOR PROPER SIGNAL AND VALVE OPERATOR	AREQUEST WAS MADE TO CHECK FOR PROPER SIGNAL AND VALVE OPERATION. VERIFIED THE STROKE ON THE MAIN REGULATOR VALVE
A28360	9 (0 FR	027	2308	09/06/84	0-FR-027-2308, TROUBLESHOOT LOOP RECORDER IS DRIVEN HARD UPSCALE.	THE LOOP RECORDER WAS DRIVEN HARD UPSCALE. REPLACED THE CIRCUIT BOARD FOR PX 3 SUFFER ON THE FLOW RECORDER CALIBRATED & RETURNED TO SERVICE
A28526	3	1	068	RVLIS		1C68-RVLIS, REMOVE RVLIS SYSTEM TRAIN A&B FROM SERVICE AND PERFORM A SAMPLE CHECK OF CARD CONNECTORS FOR HAIRLINE CRACKS PER THE ATTACHED WESTINGHOUSE TECH. BULLE	THE TRAIN ALB OF THE R.V.L.I.S. SYSTEM WERE REQUESTED BY THE SYSTEM ENGINEER TO CHECK FOR CRAKS IN THE CIRCUIT BOARDS. CHECKED FOR PROBLEMS AND FOUND NONE
A28576	1	O PX	090	SPARE	09/05/84	0-PX-090-SPARE, REPAIR XFORMER - 24VDC	THE POWER SUPPLY WAS NOT OPERATING PROPERLY. REPLACED THE TRANSISTOR AND MADE A FUNCTIONAL CHECK.
A28577	3 (O PX	090	SPARE	09/05/84	0-PX-090-SPARE, +24 VOLTS DROP TO 9 VOLTS UNDER LOAD	THE 24 VOLTS POWER SUPPLY WAS NOT SHOWING ITS APPROPRIATE. REPLACED A BAD TRANSISTOR AND VERIFIED OPERATION.
A28577	6	O PX	090	SPARE	09/06/84	0-PX-090-SPARE, REPAIR PX, FAILS UNDER LOAD	THE POWER SUPPLY WAS FAILING UNDER LOADS DUE TO A BAD CAPACITOR. REPLACED A BAD CAPACITOR & VERIFIED OPERATION.
A28577	7 (O PX	090	SPARE	09/06/84	0-PX-090-SPARE, REPAIR PX, FAILS UNDER LOAD	THE POWER SUPPLY WAS FAILING UNDER FULL LOADED CONDITION. REPLACED A REGULATOR AND A CAPACITOR AND VERIFIED OPERATION.
A28578	0 1	D PX	090	SPARE	09/06/84	0-PX-090-SPARE, REPAIR PX SUPPLY - 240DC	THE POWER SUPPLY WAS NOT OPERATING PROPERLY. FOUND A BARE WIRE WHICH WAS REPAIRED.
A28578	7 (O PX	090	SPARE	09/17/84	0-PX-090-SPARE, POWER SUPPLY MEASURES +31.6V & -23.7V INVESTIGATE AND REPAIR.	THE POWER SUPPLY WAS NOT FUNCTIONING PROPERLY. REPAIRED THE POWER SUPPLY AND RETURNED TO SERVICE.
A28599	B	ц		178	09/10/84	1-LI-063-178,1-LI-63-178 CONTAINMENT SUMP LEVEL INDICATOR READING 3% HIGHER THAN OTHER ASSOCIATED LEVEL INDICATORS	
A28602		2 FR	003	90	09/10/84	2-FR-003-90, PLEASE REPAIR BLUE PEN ON FLOW RECORDER.	THE BLUE PEN ON THE FLOW RECORDER WAS NOT PRINTING. THE BLUE PEN WAS CALIBRATED & RETURNED TO SERVICE.
A28656	4	0 811	090	SPARE	09/13/84	0-RM-090-SPARE, RP-1 HAS NO SIGNAL, REPAIR AS NEEDED.	THE POWER SUPPLY WAS NOT OPERATING PROPERLY. CHANGED OUT SEVERAL RESISTORS

								Plant	Maintenance	Summary	Page 4 of 5
	INST	RUMENT	MA	INTENANCE	MONTHLY S	SUMMARY	10-10-84	•	, PSI		
MR.COM	PU	FUNC	SYS	ADDRESS.	DATE	DESCRIPTI	ION	•••••		•••••	CORRECTIVE ACTION
											AND A CAPACITOR ALSO THE SCALE HAD WAS REALIGNED.
A287090	0 2	HZAN	043	200	09/05/84				OUT INSTRUME 43-2000 LOW		THE HYDROGEN ANALYZER WAS IN NEED OF A CHANGE OUT IN GAS BOTTLS. VERIFIED OPERABILITY AND REPLACED THE SUPPLY BOTTLE AND RETURNED TO SERVICE.
A287096	5 2	LIC	003	164	09/04/84	MAN. ON P	LVE WIDE O	ipen can Ise valvi	NOT GET CONTR	OLLER IN	THE WIRING PLUG WAS NOT PLUGGED IN PROPERLY. REPOSISTIONED THE WIRING PLUG ON THE MODULE AND LEFT IN SERVICE.
A288604	5 1	RM	090	112	09/17/84	1-RM-090-		1128 R	ADIATION DETE	CTOR FOR	THE MONITOR WAS NOT OPERATING PROPERLY. FOUND THE IODINE FLOW RATES TO BE OUT OF ADJUSTMENTS.
A28863	3 1	FCV	001			LARGE AND	DUNT OF AI	R - NEEL		IRED WHILE	REGULATOR NEEDLE STUCK INSIDE REGULATOR ORFICE. CLEANED REGULATOR & REASSEMBLED, REINSTALLED, & SET TO 45PSI.
A288664	4 1		085	88	09/25/84				t for 88 rod-		ROD BOTTOM BISTABLE LIGHT NOT ON. REPLACED BAD BISTABLE CARD.
A288706	3 1	ц	063	99	09/26/84	1-LI-063-	-99,¤I× LE	vel pegi	GED HIGH		THERE WAS A BAD CIRCUIT BOARD DUE TO AN ELECTRONIC FAILURE. REPLACED THE CIRCUIT BOARD, RECALIBRATED THE TRANSMITTER AND RETURNED TO SERVICE.
A288713	3 1	ц	003	107	09/25/84		-107,*I* 1 APPRCX. 15		07 H4 S/G LEV		THE HIGH SIDE DRAIN VALVE WAS FOUND LEAKING AND THE TRANSMITTER WAS FOUND OUT OF CALIBRATION. THE DRAIN VALVE WAS CLOSED, THE TRANSMITTER RECALIBRATED, & RETURNED TO SERVICE.
A288723	3 1	PCV	001	12		1-PCV-001 1-PCV-01-		IR LEAK	on valve pos	ITIONER ON	A GASKET WAS BAD CAUSING THE VALVE POSITIONER TO LEAK. TILE GASKET WAS REPLACED, VALVE WAS STROKED FROM CONTROL ROOM & SYSTEM RETURNED TO SERVICE.
A288733	1	PI	068	301	09/26/84	READING L		O WHILE	1-PI-68-31K		THE PRESSURE TRANSMITTER WAS FOUND OUT OF CALIBRATION DUE TO DRIFT. RECALIBRATED THE TRANSMITTER AND RETURNED TO SERVICE.
A288737	1	ц	003	107		RANGE LEV	EL INDICA	TOR READ	4 S/G CHANNEL D)/=30% LESS IND. 1-LI-3-1	THAN ITS	THE SENSE LINES TO THE TRANSMITTER WERE LOW CAUSING EMACCURATE READINGS. BACK FILLED THE SENSE LINES AND RETURNED TO SERVICE.
A288745	1		085	84	09/27/84	1085-84			S OFF AT 4-1/		ROD BOTTOM BISTABLE OUT OF CAL. RECAL ROD BOTTOM BISTABLE.

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	INSTRUMENT		MAINTENANCE		MONTHLY	SUMMARY	10-10-84		PAGE	5	
MR.COM	P U	FUNC	SYS	ADDRESS.	DATE	DESCRIP	TION		•••••		CORRECTIVE ACTION
						OUT AND RODS.	COMES ON A 2-1/2	2 STEPS W	HILE INS	ERTING	
A29275	4	2 PS	003	150A	09/14/84		3-150A, NIN CHECK D IMI-134	SWITCH W	IRING PE	R	A REQUEST WAS MADE TO CHECK THE SWITCH WIRING. SWITCH WIRING WAS VERIFIED AND RETURNED TO SERVICE. NO PROBLEM FOUND.
A29275	5	2 PS	003	1508	09/14/84		9-1509, ×I× CHECK D IMI-134	SWITCH W	IRING PE	R	CHECK THE SWITCH WIRING. VERIFIED WIRING AND RETURNED TO SERVICE. NO PROBLEM FOUND.
A29275	7	2 PS	003	165A	09/14/84		3-165A, #IN CHECK D IMI-134	SWITCH W	IRING PE	R	THE PRESSURE SWITCH WAS REQUESTED TO HAVE WIRING CHECK. CHECKED THE WIRING AND RETURNED TO SERVICE. NO PROBLEM FOUND.
A29275	8	2 PS	003	1658	09/14/84		13-1658, *I* CHECK 10 IMI-134	SWITCH W	IRING PE	R	THE PRESSURE SWITCH WAS REQUESTED TO HAVE WIRING CHECK. CHECKED THE WIRING AND RETURNED TO SERVICE. NO PROLEM FOUND.
A29316 - 34-	8	2 RM	090	120	09/04/84	2-RM-09	0-120, PERFORM SI	28A.2 ON	MONITOR	•	THE RADIATION MONITOR WAS REQUESTED TO HAVE A CALIBRATION CHECK. PERFORMED THE CALIBRATION CHECK AND RETURNED TO SERVICE.
A29316	9	2 1311	090	121	09/04/84	2-RM-09	0-121, PERFORM SI	281-2 ON	MONITOR		THE RADIATION MONITOR WAS NOT OPERATING PROPERLY. PERFORMED THE COUNT RATE ON THE RADITION MONITOR AND RETURNED TO SERVICE.
A29323	5	1 FCV	001	25	09/25/84	1-FCV-0 BLEED H	101-25, AIR REGULAT	TOR BLOWI	NG OUT D	F REG.	THE AIR REGULATOR WAS REPORTED BLOWING AIR. REMOVED THE REGULATOR DISASSEMBLED AND RETIGHTENED AND RETURNED TO SERVICE.

55 records listed.

(Continued)

Modifications Group

1. ECN 5237 - Hot Laundry

The fans were test ran and were overheating. This problem was resolved but the entire system has not been tested. Work will be discontinued until U2C2 refueling outage is completed.

2. ECN 5645 - Steam Generator Blowdown System - Unit 2

Pre-outage instrumentation work was completed. Tie-ins will be completed during U2C2 refueling outage.

3. ECN 2768 - Reactor Pressure Vessel Level Indication System (RVLIS) (Unit 2)

Variances initiated during August were completed and returned from the Office of Engineering (OE). Prefabrication work on hangers and installation was completed. The final workplan (seal table tie-in) was placed in the approval cycle.

All auxiliary building conduit work was completed this month. Preliminary work in annulus began.

4. ECN 2780 - Post Accident Sampling Facility - Unit 2

Minor HVAC hanger work, sealing of the enclosure and miscellaneous tubing work was completed. Preop testing on the HVAC began.

The sampling system is nearing completion, with several devices energized and undergoing functional testing.

5. ECN 6055 - Wide Range Pressure Transmitter to the RVLIS Panel

Prefabrication activities were completed.

6. ECN 5194 - Iodine Monitoring Building

The mechanical work was completed.

The sampling and air conditioning systems were turned over for vendor functional testing.

7. ECN 5024 - Lay-up Water Treatment

Prefabrication activities required for the U2C2 refueling outage were completed. Fifteen of the seventy operator valves are completed.

8. ECN 5009 - ERCW Piping Change-Out

All of the prefabrication work that could be done was completed.

(Continued)

Modifications Group

(Continued)

9. NUREG 0588

5895/Solenoids The accessible solenoids valves on this ECN were changed out. The remaining work will be completed during U2C2 refueling outage. 6032/H₂ Analyzer Relocation All pre-outage work was completed. Final relocation will be completed during the U2C2 refueling outage. 5883/Pressure Switch All pre-outage work was completed Final Relocation relocation will be completed during the U2C2 refueling outage. 6200/Pressure transmitter All pre-outage work was completed. Final Relocation relocation will be completed during the

10. ECN 6206 - Paint CST "B"

The workplan for this outage activity was written and approved.

11. ECN 5373 - Condensate Demin. System Mod.

The final design utilizing raw cooling water instead of ERCW was approved by Office of Engineering. The workplan was revised and submitted for approval.

U2C2 refueling outage.

12. ECN 5470 - ABGTS F.P. Nozzle Replacement

Prefabrication activities were started on this work.

13. ECN 5626 - Annulus Ladder

A workplan to reroute a piping interference during the U2C2 refueling outage was written and submitted for approval.

14. ECN 5631 - Hand Rails at CCW Pump Station

This work was started and is approximately 50% complete. It will be placed in hold until after the U2C2 refueling outage.

15. ECN 5773/6196 - PZE HGR MODS

Several meetings were held regarding the final fix. The current plan is to modify nine supports plus as many of the remaining 30 as possible during the U2C2 refueling outage. A workplan for the first nine was written and submitted for approval.

(Continued)

Modifications Group

(Continued)

16. L DCR 1739 - VAACS Computer Installation

Conduit, cable and material installation continued.

17. ECN 5198 - Technical Support Center

All outside conduit work was completed, cable pulling is 80% complete. P250 - DPM 50 intertie is completed to the point of functional testing and termination of the signal cable.

18. ECN 5604 - Wells Fargo Alarm

All but one set of alarms are ready for functional testing.

19. Evacuation Alarms

Unit valve room and the letdown heat exchanger room is completed. Work will be suspended until after the refueling outage. Estimated eight weeks of work remains.

20. ECN 5770 - Rad Mod Condenser Exhaust

The monitor was completed and turned over for functional testing.

21. ECN 5846 - Remove Relay 62 from Shutdown BD Breakers

The last breaker was completed this month.

22. ECN 5865 - Relocate LA-77-124

Approximately 90% of the conduit is installed.

23. ECN 5867 - Fuel Transfer System

Both upenders have been functional tested. Remaining work is the checkout of the transfer cart and setting the limits.

24. ECN 5882 - Temperature Switch Replacement

Conduit, cable and instrument installation has started.

- 25. ECN 5883 Replacement of Flow Switches Conduit work commenced this period.
- 26. ECN 6832 Relocate H₂ Analyzer

Conduit and cable pulls are 80% complete.

TENNESSEE VALLEY AUTHORITY

Sequoyah Nuclear Plant P. O. Box 2000 Soddy-Daisy, Tennessee 37379

October 15, 1984

Nuclear Regulatory Commission Office of Management Information and Program Control Washington, DC 20555

Gentlemen:

Enclosed is the September 1984 Monthly Operating Report to the NRC for Sequoyah Nuclear Flant.

Very truly yours,

TENNESSEE VALLEY AUTHORITY

P.R. Walls

P. R. Wallace Plant Manager

Enclosure cc (Enclosure): Director, Region II Nuclear Regulatory Commission Office of Inspection and Enforcement 101 Marietta Street Suite 3100 Atlanta, GA 30323 (1 copy)

> Director, Office of Inspection and Enforcement Nuclear Regulatory Commission Washington, DC 20555 (10 copies)

> Mr. A. Rubio, Director Electric Power Research Institute P. O. Box 10412 Palo Alto, CA 94304 (1 copy)

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