

OCT 11 1984

ENCLOSURE 1

NOTICE OF VIOLATION

Duke Power Company  
McGuire Nuclear Station

Docket Nos. 50-369, 370  
License Nos. NPF-9, 17

The following violation was identified during an inspection conducted on June 20 - July 20, 1984. The Severity Level was assigned in accordance with the NRC Enforcement Policy (10 CFR Part 2, Appendix C).

Technical Specification 6.8.1 requires that written procedures be established and implemented covering surveillance testing of safety-related equipment. Implicit in that requirement is that the procedures be adequate. Also, this Technical Specification requires that applicable procedures implement the requirements of NUREG-0737. Item I.C.6, Operating Activities, in effect, requires independent verification so as to verify correct performance of operating activities.

Contrary to the above, certain surveillance procedures and drawings were inadequate and failed to incorporate independent verification in that the following conditions were noted:

- a. On April 17, 1984, following the completion of surveillance test procedure PT/1/A/4700/27, Containment Spray Check Valve Inservice Test, the licensee failed to reclose a vent valve located in the auxiliary containment spray system. Procedure PT/1/A/4700/27 was inadequate in that there was no procedure step for restoration of the valve and no independent verification to ensure that it was returned to its normal closed position following valve manipulation. From April 17 through June 27, 1984, the vent valve remained open and the Unit 1 reactor was critical during the majority of that time, resulting in a potential loss of containment integrity. (Radiological consequences and system performance during accident conditions were evaluated with the vent valve being open in which it was found to have minimal impact on its design function).
- b. On July 3, 1984, while trouble shooting the cause of a failed indicator light of the Unit 2 main steam isolation valve during a routine surveillance of relay circuitry, the technician used an inadequate drawing and erroneously lifted a lead in a normal current path which resulted in a reactor trip.

This is a Severity Level IV violation (Supplement I).

Pursuant to 10 CFR 2.201, you are required to submit to this office within 30 days of the date of this Notice, a written statement or explanation in reply, including: (1) admission or denial of the alleged violations; (2)

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the reasons for the violations if admitted; (3) the corrective steps which have been taken and the results achieved; (4) corrective steps which will be taken to avoid further violations; and (5) the date when full compliance will be achieved.

Security or safeguards information should be submitted as an enclosure to facilitate withholding it from public disclosure as required by 10 CFR 2.790(d) or 10 CFR 73.21.

**OCT 11 1984**

Date: \_\_\_\_\_