

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) DOCKET NUMBER (2) PAGE (3)
 LaSalle County Station Unit 1 0 5 0 0 0 3 7 B 1 OF 0 3

TITLE (4)
 Containment B & C Tests Exceed Tech Spec Limit

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER (5)
10	04	84	84	064	00	11	02	84			0 5 0 0 0
DOCKET NUMBER (5): 0 5 0 0 0											

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)

OPERATING MODE (8) 4	20.402(b)	20.408(a)	50.73(a)(2)(iv)	73.71(b) 73.71(e) OTHER (Specify in Abstract below and in Text, NRC Form 386A)
	20.408(a)(1)(i)	50.35(a)(1)	50.73(a)(2)(iv)	
	20.408(a)(1)(ii)	50.35(a)(2)	50.73(a)(2)(iv)	
	20.408(a)(1)(iii)	50.73(a)(2)(ii)	50.73(a)(2)(iv)(A)	
	20.408(a)(1)(iv)	X 50.73(a)(2)(ii)	50.73(a)(2)(iv)(B)	
	20.408(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(v)	

LICENSEE CONTACT FOR THIS LER (12)

NAME Robert D. Koenig, extension 575	TELEPHONE NUMBER	
	AREA CODE 8 1 5	3 5 7 - 6 7 6 1

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC
B	SJ	IISV	A3191	Y	X	SJ	IISV	A3191	Y
X	BO	IISV	A3191	Y					

SUPPLEMENTAL REPORT EXPECTED (14)

<input checked="" type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)	<input type="checkbox"/> NO	EXPECTED SUBMISSION DATE (15)
		MONTH DAY YEAR 1 2 1 5 8 4

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-spaced typewritten lines) (16)

On October 4, at 0500 hours, while performing LTS-100-9 (Feedwater and RWCU Return Valve Leak Rate Test Preparation) on Unit 1, it was discovered that valve 1B21-F010A (Feedwater Inboard Check Valve) was not fully seated. When a Type C LLRT was performed, it was found that the containment leakage limit for Type B & C leakage had been exceeded (0.6 La). The leakage rate was above 2100 SCFH; the limit is 231.4 SCFH.

The hinge pin bushing on one side of the check valve disc had moved out of the disc. This prevented the valve from fully closing. The bushing was subsequently replaced and pressed into the disc (0.001 inch, press fit), to prevent any bushing "moving-out".

Valves 1B21-F032B and 1E51-F064 were also found to exceed the allowable Technical Specification limits. The valves will be repaired prior to the unit's restart.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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		YEAR 84	SEQUENTIAL NUMBER -064-	REVISION NUMBER 00	02	OF 03

TEXT (If more space is required, use additional NRC Form 365A's) (17)

I. EVENT DESCRIPTION

On October 4, 1984, at 0500 hours, while performing LTS-100-9 ("Feedwater (SJ) and RWCU (CE) Return Valve Leak Rate Test Preparation") on Unit 1, it was discovered that valve 1B21-F010A (Feedwater Inboard Isolation Check Valve) was not fully seated. This valve is a primary containment (NH) isolation valve and is required to close in the event of a loss of coolant accident (LOCA). When a type "C" local leakage rate test was performed, a leakage rate of greater than 2100 SCFH (Standard Cubic Feet per Hour) was found. This leakage rate placed the total Type B and C leakage for the Unit 1 containment above the limit of 0.6 La, 237.4 SCFH (Technical Specification 3.6.1.2.b).

Since this valve has failed, other valves have had the magnitude of leakage which would have placed the containment Type B & C leakage rate above 0.6 La. Valve 1B21-F032B, outboard feedwater check valve, was found to have a leakage rate of approximately 150 SCFH, and valve 1E51-F064, RCIC (BN) Steam Supply to RHR (BO) Steam Condensing, was found to have a leakage rate of approximately 150 SCFH.

II. CAUSE

The cause of 1B21-F010A not fully closing was the hinge pin bushing becoming wedged between the disc anti-rotation weld and the hinge pin shoulder. By design, the hinge pin bushing is press fit into the disc bore. The anti-rotation weld is applied to the disc to prevent the hinge pin bushing from rotating relative to the disc during power operations. It appears the hinge pin bushing to disc bore press fit was not maintained such that the bushing was able to move out of the disc bore during power operations. Once sufficiently out of the disc bore, the bushing was free to rotate relative to the disc. The hinge pin bushing shoulder subsequently become wedged between the anti-rotation weld and the hinge pin shoulder. This wedge prevented the valve from fully closing.

The causes for the 1B21-F032B and 1E51-F064 failures have not been determined as of this date.

III. PROBABLE CONSEQUENCES OF THE OCCURRENCE

The feedwater lines have redundant isolation valves. In the event of a LOCA the feedwater line would have also been isolated by feedwater isolation valves 1B21-F065A/B and 1G33-F04C. These isolation valves would limit the containment leakage from feedwater to well below Technical Specification limits.

The RCIC line has redundant isolation valves. Also, valves 1E51-F076 and 1E51-F063 would have isolated the RCIC penetration in the event of a LOCA.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
		84	064	0	03	OF 03

TEXT (If more space is required, use additional NRC Form 366A's) (17)

IV. CORRECTIVE ACTIONS

The bushings in the 1B21-F010A feedwater check valve were replaced. The new bushings were pressed into the disc body (0.001 inch, press fit) to prevent any moving out of the bushing. A Local Leak Rate Test has been performed on the 1B21-F010A since repairs, and found to be well below limits (4.42 ± 0.05 SCFH). The 1B21-F010B was also tested during this outage as satisfactory. Even though it is felt that this is an isolated case, the Unit 2 valves, 2B21-F010A and B, will be leak rate tested at the next planned outage of sufficient duration. (AIR 1-84-247)

Valves 1B21-F032B and 1E51-F064 have not been repaired as of this date. Work Request numbers L43041 and L42216 have been written to repair these valves. A supplemental report (AIR 1-84-67169) will be issued upon completion of valve repairs. In addition, if any other valves are found to have a leakage rate placing the total B & C leakage rate above 0.6 La, then these valves will be reported in the supplemental report. Other valves which had leakage above the LaSalle Administrative limit, but whose "as found" leakage rates did not exceed the Technical Specification limits, will also be repaired during the outage.

V. PREVIOUS OCCURRENCES

Several failures of the feedwater check valves (failure to pass Type "C" tests) have occurred on Unit 1. (LER's 373/83-146/03X-1 and 373/84-012-00).

VI. NAME AND TELEPHONE NUMBER OF PREPARER

R. D. Koenig, 815/357-6761, extension 575.



Commonwealth Edison
LaSalle County Nuclear Station
Rural Route #1, Box 220
Marseilles, Illinois 61341
Telephone 815/357-6761

November 2, 1984

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Dear Sir:

Reportable Occurrence Report #84-064-00, Docket #050-373 is being submitted to your office in accordance with 10CFR 50.73.

G. J. Diederich
G. J. Diederich *11/2/84*
Superintendent
LaSalle County Station

GJD/MLD/kg

Enclosure

xc: NRC, Regional Director
INPO-Records Center
File/NRC

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