NRC Form 366 19-83)						LICENSEE EVENT REPORT (LER)									U.S. NUCLEAR REGULATORY COMMISSION APPROVED OMB NO. 3150-0104 EXPIRES. 8/31/85															
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COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE SYSTEM

COMPONENT

AREA CODE

MANUFAC TURER

EXPECTED

817 | 01 - 16 | 614 | 4

YEAR

DAY

REPORTABLE TO NPROS

MONTH

SUPPLEMENTAL REPORT EXPECTED (14)

ABSTRACT /Limit to 1400 spaces, i.e. approximately fifteen single space typewritten lines) (16)

Michael R. Cooper, Compliance Section Engineer

TO NEBDS

MANUFAC TURER

Following additional inspections of various safety-related systems at Sequoyah, interactions were found that were not in compliance with Appendix R of 10 CFR 50. Fire watches had already been established in this area due to commitments already made by TVA dealing with Appendix R. This fire watch satisfies requirements per action statement of Technical Specification 3.7.12 and will remain in effect till full compliance with Appendix R can be achieved. This report is required per license condition 2.H, 10 CFR 50.73 (a)(2)(ii) and special report requirements of Technical Specification 3.7.12.

There was no effect on public health or safety.

Previous occurrences - five - SQRO-50-327/84046, SQRO-50-327/84049, SQRO 50-327/84051, SQRO-50-327/84057, and SQRO-50-327/84059.

8411130638 841101 PDR ADOCK 05000327 PDR

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CAUSE SYSTEM

COMPONENT

YES III yes, complete EXPECTED SUBMISSION DATE!

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104 EXPIRES: 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)			LE	R NUMBER (6)	PAGE (3)				
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Sequoyah, Unit 1	0 5 0 0 0 3 2	7 8 1	4	_	0 6 3	_	0 0	0	2 OF	0 3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

Inspections at Sequoyah Nuclear Plant have identified the following items of non-compliance with Appendix R of 10 CFR 50. These inspections are part of an ongoing project to ensure compliance with Appendix R.

- 1. The listed train 'A' cables interact with unit 2 2B 480 volt transformer room (train 'B').
 - Train 'A' cables: 2V2233, 2V2232A (2-FCV-62-98), 2V2070A, 2V2071A (2-LCV-62-135), 2V2326A, 2V2327A (2-FCV-63-7), 2V2680A, 2V2681A (2-FCV-63-26), 2V2695A, 2V2696A (2-FCV-63-39), 2V2292A, 2V2993A (2-FCV-70-143).
- 2. The listed train 'A' cables interact with unit 2 train 'B' MOV board room.
 - Train 'A' cables: 2PL3002A (CCP Room Cooler A-A), 2PL4729A (CCS Pump 2A-A), 2PL6145A, 2PL6148A, 2PL6149A (CCP 2A-A Auxiliary Oil Pump), 2V2235A (2-FCV-62-98), 2V2073A (2-LCV-62-135), 2V2329 (2-FCV-63-7), 2V2683A (2-FCV-63-26), 2V2698A (2-FCV-63-39), 2V2787A (2-FCV-74-1), 2V2295A (2-FCV-70-143), 2V2423A, 2V2424A, 2V2425A (2-FCV-67-146).
- 3. The listed train 'B' cables interact with train 'A' 2A 480 volt transformer room.
 - Train 'B' cables: 2V2240B, 2V2241B (2-FCV-62-99), 2V2100B, 2V2101B (2-LCV-62-136), 2V2320B (2-FCV-63-6), 2V4934B (2-FCV-63-24), 2V2840B, 2V2842B (2-FCV-74-2), 2V4371B (2-FCV-62-55), 2PM-132 (2-FCV-62-56).
- 4. Interactions exist for each channel such that all four RCS temperature indicators could be made inoperable due to one fire. Cables involved are:
 - Channel I: 2PM591I (T-68-1), 2PM778I (T-68-18), 2PM686I (T-68-24), 2PM871I (T-68-41).
 - Channel II: 2PM595II (T-68-43), 2PM784II (T-68-60), 2rM69III (T-68-65), 2PM876II (T-68-63).
- 5. Unit 1 RCS pressure instrumentation loop P-68-66 (cables 1PM481II, 1PV135II) interacts with pressure instrumentation loop P-68-69 (cables 1PM480I, 1PV320J) at three locations in auxiliary building (elevation 690 U-W/A3-A4, elevation 714 Q-W/A2-A4, elevation 714 Q-R/A4-A11). In addition to cables, the pressure transmitting equipment is located less than 20-feet from each other at elevation 690 at A3-A4/V line.
- 6. Secondary side pressure control cables for all three channels of pressure indication for each steam generator interact with less than 20-foot separation. Also cables for 2-L-381 pressure indicator are involved. Cables involved are:
 - SG-1: 2PM1335I (PI-1-2A), 2PM1347II (PI-1-2B), 2PM1360IV (PI-1-5).
 - SG-2: 2PM1474I (PI-1-9A), 2PM1480II (PI-1-9B), 2PM1490III (PI-1-12).
 - SG-3: 2PM1595I (PI-1-20A), 2PM1607II (PI-1-20B), 2PM1613III (PI-1-23).
 - SG-4: 2PM1715I (PI-1-27A), 2PM1723II (PI-1-27B), 2PM1729IV (PI-1-30).
 - 2-L-381: 2PM1340J, 2PM1475J, 2PM1601J, 2PM1717J.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104 EXPIRES: 8/31/85

FACILITY NAME (1)	DOCKET NUMBER)	DOCKET NUMBER					LER NUMBER (6)									
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TEXT (If more space is required, use additional NRC Form 366A's) (17)

- 7. Interaction exists in the 480 volt reactor MOV board rooms between 1A-A and 2A-A 480 volt reactor MOV board room air handling units (cables 1PL2958A and 2PL2958A) and numerous cables for the 1B-B and 2B-B 480 volt reactor MOV board room air handling units and compressors.
- 8. The following interactions exist for 480 volt reactor MOV board rooms at elevation 734 in the auxiliary building.
 - a. Cable 1PL2958A (AHU 1A-A) with numerous cables for 1B-B AHU/compressor.
 - b. Cable IPL2947V (1B-B compressor) crosses R line wall and interacts with 'A' train 6.9 KV shutdown boards.
 - c. Cable 2PL4947B (2B-B compressor) interacts with numerous cables for the 2A-A air handling unit/compressors. Also cable 2PL2958A crosses R line into 'B' train 6.9 KV shutdown board room.
- 9. At elevation 749 in the 2B 480 volt MOV board room in the auxiliary building, cables 2PL2945A, 2960A, 2961A, 3503A, 3504A, 3505A (2A board room AHU/compressors) interact with cable 2PL2947B (2B compressor).
- 10. At elevation 749 in the auxiliary building (2A 480 volt transformer room, 2A 480 volt HVAC equipment room) cables 2PL2966B, 2PL3514B, 2PL3516B pass through two rooms listed above and interact with transformers/HVAC equipment on train 2A.
- 11. A fire in auxiliary building at elevation 690 All-Al5/Q-S would result in loss of automatic level control for the turbine-driven auxiliary feedwater pump and the wide range level indication cables (2PM1233K, 2PM1244K, 2PM1226K) required for manual operation of level control valve.
- 12. At elevation 669 in auxiliary building at All-Al2/S-T the DC control power for the turbine-driven auxiliary feedwater pump cable 2SG220A interacts with the 'A' and 'B' motor-driven auxiliary pump on elevation above, due to stairway opening.

The action statement for Technical Specification 3.7.12 was satisfied by utilizing fire watches in the affected areas that were established by other Appendix R commitments. This action included the establishment of a roving fire watch in areas with fire detection and a dedicated fire watch in areas without fire detection.

An implementation schedule for corrective actions will be submitted to the NRC by January 1, 1985. The fire watches established will remain in effect until full compliance with the requirements of Appendix R can be achieved.

TENNESSEE VALLEY AUTHORITY

Sequoyah Nuclear Plant Post Office Box 2000 Soddy Daisy, Tennessee 37379

November 1, 1984

U.S. Nuclear Regulatory Commission Document Control Desk Washington, DC 20555

Gentlemen:

TENNESSEE VALLEY AUTHORITY - SEQUOYAH NUCLEAR PLANT UNIT 1 - DOCKET NO. 50-327 - FACILITY OPERATING LICENSE DPR-77 - REPORTABLE OCCURRENCE REPORT SQR0-50-327/84063

The enclosed licensee event report, Special Report, and Facility Operating License Required Report per Section 2.H provide details concerning items of noncompliance with Appendix R of 10 CFR 50. This event is reported in accordance with 10 CFR 50.73, paragraph (a)(2)(ii).

Very truly yours,

TENNESSEE VALLEY AUTHORITY

P. R. Wallace Plant Manager

Enclosure cc (Enclosure):

James P. O'Reilly, Director U.S. Nuclear Regulatory Commission Suite 2900 101 Marietta Street, NW Atlanta, Georgia 30323

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Suite 1500
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Atlanta, Georgia 30339

NRC Inspector, NUC PR, Sequoyah

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