NARRATIVE REPORT
UNIT 1

September 1st	0000	Unit off-line due to reactor scram on loss of condenser vacuum on 8-31-84.
	0348	Unit tied to line & increasing power.
	0800	Vacuum dropping again. Power being held at approximately 20% of rated capacity.
September 3rd	0900	Began pulling rods & ramping up from loss of condenser vacuum derating.
September 4th	2400	Unit at maximum power via 105% recirc flow. Unit also in fuel cycle coastdown which is now approximately 85% of design capacity.
September 28th	2100	Reducing load for scheduled refueling outage.
September 29th	0008	Turbine tripped and unit off-line for 10 week refueling outage.

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HATCH 1 SAFETY-RELATED MAINTENANCE REQUESTS
TO BE REPORTED FOR September 1984

NUMBER	DATE COMPLETED	DESCRIPTION
82-2143	08-28-84	Ran weld leads for torus work thru penetration sleeve on east reactor bldg. wall-south of column R3. Ref. DCR 80-199.
82-2729	09-15-82	Installed drains & vent lines, & sump pump to new shift supervisor's office/kitchen in control bldg. (MPL 1Z44). Ref. DCR 80-348.
84-1550	09-11-84	Installed 3/4" air compressor pressure switch root valve into existing 2 1/2" JBE process line. (MPL # 1R43-S001B). Ref. DCR 80-98.
84-2189	08-20-84	Replaced existing Standby Liquid Control Relief Valve (MPL 1C41-F029A) Ref. DCR 84-235.
84-3232	07-26-84	Cut barstock from plate in warehouse and transferred heat number to each piece of bar. (MPL 1T23) Ref. DCR 82-75.
84-4524	08-06-84	Removed existing SRV Tailpipe Temperature Recorders (L&N Speedomax) and installed new recorder (L&N Series 250) (MPL 1B21-R614) Ref. DCR 84-001.

OPERATING DATA REPORT

DOCKET NO. 50-321
DATE 10-10-84
COMPLETED BY: M.G. McBay
TELEPHONE (912) 367-7851

OPERATING STATUS

Notes

1. Unit Name: E. I. Hatch Nuclear Plant Unit 1
2. Reporting Period: 09-84
3. Licensed Thermal Power (MWt): 2436
4. Nameplate Rating (Gross MWe): 809.3
5. Design Electrical Rating (Net MWe): 777.3
6. Maximum Dependable Capacity (Gross MWe): 801.2
7. Maximum Dependable Capacity (Net MWe): 752.2
8. If Changes Occur in Capacity Ratings (Items Number 3 through 7) Since Last Report, Give Reasons:
9. Power Level to Which Restricted, If Any (Net MWe)
10. Reasons for Restrictions, If Any:

	This Month	Yr-to-Date	Cumulative
11. Hours In Reporting Period	720	6575	76702
12. Number of Hours Reactor was Critical	672.1	5638.8	55164
13. Reactor Reserve Shutdown Hours	0	0	0
14. Hours Generator On-Line	668.3	5474.9	51867.9
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy Generated (MWH)	1294200	12151162	109369190
17. Gross Electrical Energy Generated (MWH)	398050	3797550	35255720
18. Net Electrical Energy Generated (MWH)	375747	3609181	33468544
19. Unit Service Factor	92.8	83.3	67.6
20. Unit Availability Factor	92.8	83.3	67.6
21. Unit Capacity Factor (Using MDC Net)	69.4	73.0	58.0
22. Unit Capacity Factor (Using DER Net)	67.1	70.6	56.1
23. Unit Forced Outage Rate	0	15.0	18.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
Refueling 10-1-84 10 week duration

25. If Shutdown at End of Report Period, Estimated Date of Startup:
26. Units in Test Status (Prior to Commercial Operation):

	Forecast	Achieved
INITIAL CRITICALITY		
INITIAL ELECTRICITY		
COMMERCIAL OPERATION		

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-321
DATE: 10-10-84
COMPLETED BY: M.G. McBay
TELEPHONE (912) 367-7851

MONTH 09-84

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	106	17	611
2	142	18	607
3	309	19	606
4	546	20	603
5	651	21	601
6	630	22	596
7	630	23	593
8	628	24	591
9	625	25	589
10	623	26	588
11	618	27	587
12	615	28	554
13	612	29	-15
14	609	30	-15
15	607	31	
16	612		

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UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-321
UNIT NAME Hatch 1
DATE 10-4-84
COMPLETED BY M. McBay
TELEPHONE 912-367-7851

REPORT MONTH September

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report #	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
84-68	840901	F	3.8	A	2	NA	HC	XXXXXXXX	Loss of condenser vacuum from in-leakage through crack between condenser & LP turbine. Outage
84-69	840901	F	53.2	B	5	NA	HC	XXXXXXXX	Load at approximately 20% due to instability of vacuum. Problems being resolved.
84-70	840903	F	39.0	F	5	NA	HC	XXXXXXXX	Ramping up from vacuum problems to rated via recirc flow.
84-71	840928	S	3.13	C	2	NA	RC	FUELXX	Reducing load for scheduled refueling outage.
84-72	840929	S	47.87	C	2	NA	RC	FUELXX	Unit refueling outage in progress.

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Continuations
5-Load Reduction
9-Other (Explain)

4
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

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Exhibit I - Same Source

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NARRATIVE REPORT
UNIT 2

September 1st	0000	Unit still in recirc piping replacement outage.
September 3rd	0758	Reactor mode now switched to run. Prepared for unit startup.
	1817	Turbine tripped.
	1837	Turbine reset.
	2151	Unit on line at approximately 50 GMWe.
September 4th	0140	Main turbine tripped due to MSR hi level.
	0335	Main turbine back on line following MSR hi level trip.
	0401	Main turbine tripped due to MSR hi level.
	0552	Main turbine back on line following MSR hi level trip.
	0617	Main turbine tripped due to MSR hi level.
September 5th	0122	Turbine rolling. Prepared to tie unit on line.
	0232	Unit now tied on line following multiple turbine trips. Startup testing in progress. Power at approximately 20% of rated.
September 6th	2300	Pulling rods to approximately 40% CMWt to increase power. Startup testing still in progress.
September 8th	0000	Holding power at approximately 70% for startup testing.
September 10th	0000	Still holding power at approximately 70% for testing.

September 11th	0132	Both recirc pumps tripped during procedure testing (HNP-2-10263).
	0545	Back to 70% power from recirc pump trip & holding.
	2122	Inserting rods for testing of scoop tubes on recirc pumps.
	1939	Testing scoop tubes complete. Commencing to pull rods per STA instructions to rated power.
September 15th	0535	Rod pulling stopped due to detected problems with condensate demineralizers.
	0715	Reducing reactor back to approximatey 630 MWe.
	0735	Stopped reducing load.
	1110	2E condensate demineralizer taken out of service because of low flow rate & high resin trap.
September 15th	2256	2E condensate demineralizer placed backed in service after backwash & precoating. Increasing power again.
September 17th	2150	Reducing power for the removal of condensate demineralizer 2E. Power reduced to approximately 75% of rated.
September 20th	0320	Increasing power via recirc flow & beginning preconditioning ramp to rated power.
September 21st	1701	Unit at rated power. Reactor scram from the inboard MSIV's drifting closed.
September 22nd	1603	Reactor critical. Preparing to put unit on line.
September 23rd	1647	Main generator on line. Unit again going through startup.
September 24th	1040	Holding load at approximatey 60% of rated to complete OD-1.
	1614	OD-1 completed.

September 24th	1721	Condensate booster pump trip when auxiliary oil pump switched to auto position.
	2205	Recirc pumps placed in master manual position. Begin increasing power.
September 25th	1858	Holding load for RHR pump bolt problems.
September 26th	1910	"B" recirc pump drifting down. Load reduced down to approximately 45% of rated.
September 27th	1245	Increasing reactor power via rod pulls & recirc flow following recirc pump problem. Increasing to rated.
September 30th	0007	Unit running at rated power. Load reduced during startup testing procedure (HNP-2-10263).
	0600	Unit operating at 2436 CMWt.
	1145	Reactor scram due to a loss of condenser vacuum. Off line at end of reporting period.

HATCH 2 SAFETY-RELATED MAINTENANCE REQUESTS
TO BE REPORTED FOR September 1984

NUMBER	DATE COMPLETED	DESCRIPTION
83-2896	02-03-84	Retagged panel valves and tag the instrument valves on 2H21 instrument racks. Ref. DCR 81-139, Rev. 1.
83-4948	01-13-84	Installed cable raceways on Reactor Protection System & scram discharge volume. (MPL 2C11). Ref. DCR 82-220.
83-6142	04-09-84	Pulled cables per HNP-6921 on the drywell pneumatics system. (MPL 2P70). Ref. DCR 81-112.
84-203	08-26-84	Redlined, functional tested, and calibrated all instrumentation and associated logic installed on the ATTS. (MPL 2A70) Ref. DCR 81-139, Rev. 1.
84-477	05-31-84	Installed replacement valve for existing 2P70-F076H D/W pneumatic valve. Ref. DCR 84-152.
84-675	08-09-84	Modified piping supports 2G41-FPC-R134 & H81. Ref. DCR 82-257.
84-678	08-09-84	Modified pipe supports on instrument air system (MPL 2P52) Ref. DCR 82-257.
84-724	07-17-84	Modified pipe support 2T46-SBG-A47. Ref. DCR 82-257.
84-726	07-02-84	Modified pipe supports on RHR system (MPL 2E11). Ref. DCR 82-257.
84-906	07-02-84	Modified existing pipe support 2P41-SW-HR117. Ref. DCR 82-257, Rev. 1.
84-915	08-09-84	Modified existing pipe supports for 2C11 system. Ref. DCR 82-257.

HATCH 2 SAFETY-RELATED MAINTENANCE REQUESTS
TO BE REPORTED FOR September 1984

NUMBER	DATE COMPLETED	DESCRIPTION
84-1020	07-23-84	Modified pipe supports 2P52-S31-H802, H804, H806 & 2P52-S43-H804. Ref. DCR 82-257.
84-1482	07-02-84	Modified existing piping supports for 2G31 system. Ref. DCR 82-257.
84-1486	07-23-84	Modified pipe supports on 2P41 system. Ref. DCR 82-257.
84-1773	07-17-84	Removed existing flow instruments (2C11-R602A,B) & installed new Rosemount transmitters 2E11-N019A&B. Also removed orifice plates and installed new orifice plates. 2E11-N006-A&B. Ref. DCR 82-158.
84-2012	07-08-84	Installed supports, piping & tubing for instruments 2G31-N031A&B. Installed Roto Hammer Gearbox reach rods on valve 20A and also gear box supports. Made core drill for reach rod assembly for valve 2G31-F208A & sealed abandoned penetration. Ref. DCR 83-86.
84-2022	08-26-84	Tagged/retagged instrument panel (isolation & drain) valves & instrument valves for Analog Transmitter Trip System instrument racks (MPL 2H21) Ref. DCR 81-139, Rev. 1.
84-2453	08-20-84	Redlined Primary Containment Cooling System circuitry (MPL 2T47-B008A&B) Ref. DCR 83-173.
84-2510	08-21-84	Installed & calibrated strain gauges on startup instrumentation 2B31-G001. Ref. DCR 83-173.

HATCH 2 SAFETY-RELATED MAINTENANCE REQUESTS
TO BE REPORTED FOR September 1984

NUMBER	DATE COMPLETED	DESCRIPTION
84-2560	07-23-84	Modified hanger 2B21-MSRV-R60 at Main Steam Relief Valve. Ref. DCR 84-044.
84-2996	07-05-84	Investigated leak at RWCU F/D "B" Resin Trap Isolation Valve (2G31-F063B). Ref. MR 2-84-2779.
84-3132	07-12-84	Terminated cables & performed internal wiring in panels to implement required logic changes in Standby Gas Treatment & Ventilation system circuits. (MPL 2C61-K48A & 49A). Ref. DCR 81-139, Rev. 1.
84-3989	08-20-84	Replaced existing motor pinion, wornshaft gear & torque switch limiter plate with new parts on RHR Torus Isolation Valves (MPL 2E11-F028B). Ref. DCR 84-211.
84-4144	08-24-84	Modified existing reactor bldg. cable tray/conduit supports per 10CFR21 seismic evaluation. (MPL 2R33). Ref. DCR 84-227.
84-4146	08-18-84	alibrated & checked setpoint on 2E32-N655 Bailey Alarm Unit Setpoint changed from 2.40 volts DC to 2.26 volts DC Ref. DCR 83-10.
84-4256	08-21-84	Replaced existing 25 amp circuit breakers in MCC 2R24-S012 component 13B & 13C with 30 amp circuit breakers for MCVs 2E21-F004B & 2E21-F005B. Ref. DCR 81-177.
84-4452	08-21-84	Installed new conduit supports & added anchor bolt to existing conduit supports. (MPL R33) Ref. DCR 80-348, Rev. 2.

OPERATING DATA REPORT

DOCKET NO. 50-366
DATE 10-10-84
COMPLETED BY: M.G. McBay
TELEPHONE (912) 367-7851

OPERATING STATUS

Notes

1. Unit Name: E. I. Hatch Nuclear Plant Unit 2
2. Reporting Period: 08-84
3. Licensed Thermal Power (MWt): 2436
4. Nameplate Rating (Gross MWe): 817.0
5. Design Electrical Rating (Net MWe): 784.0
6. Maximum Dependable Capacity (Gross MWe): 803.9
7. Maximum Dependable Capacity (Net MWe): 747.9
8. If Changes Occur in Capacity Ratings (Items Number 3 through 7) Since Last Report, Give Reasons:
9. Power Level to Which Restricted, If Any (Net MWe):
10. Reasons for Restrictions, If Any:

	This Month	Yr-to-Date	Cumulative
11. Hours In Reporting Period	720	6575	44328
12. Number of Hours Reactor was Critical	685	1059.4	28130.7
13. Reactor Reserve Shutdown Hours	0.0	0	0
14. Hours Generator On-Line	566.1	874.3	26662.2
15. Unit Reserve Shutdown Hours	0.0	0	0
16. Gross Thermal Energy Generated (MWH)	986668	1715218	56931473
17. Gross Electrical Energy Generated (MWH)	312780	555420	18727200
18. Net Electrical Energy Generated (MWH)	294596	504981	17794088
19. Unit Service Factor	78.6	13.3	60.1
20. Unit Availability Factor	78.6	13.3	60.1
21. Unit Capacity Factor (Using MDC Net)	54.7	10.3	53.7
22. Unit Capacity Factor (Using DER Net)	52.2	9.8	51.2
23. Unit Forced Outage Rate	12.9	8.8	13.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
Recirc piping outage still in progress.

25. If Shutdown at End of Report Period, Estimated Date of Startup:
26. Units in Test Status (Prior to Commercial Operation):

	Forecast	Achieved
INITIAL CRITICALITY		
INITIAL ELECTRICITY		
COMMERCIAL OPERATION		

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-366
DATE: 10-10-84
COMPLETED BY: M.G. McBay
TELEPHONE (912) 367-7851

MONTH 09-84

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	-15	17	640
2	-17	18	585
3	-13	19	591
4	-12	20	700
5	105	21	544
6	204	22	-17
7	362	23	9
8	538	24	369
9	521	25	670
10	522	26	650
11	476	27	454
12	276	28	708
13	503	29	765
14	639	30	334
15	582	31	
16	602		

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UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-366
UNIT NAME Hatch 2
DATE 10-4-84
COMPLETED BY M. McBay
TELEPHONE 912-367-7851

REPORT MONTH September

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report #	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
84-5	840901	S	69.85	H	2	NA	CB	PIPEXX	Recirc pipe replacement outage.
84-6	840903	F	28.41	A	9	NA	HA	VESSEL	Turbine trips during startup due to MSR Hi Level. Unit off-line majority of event duration.
84-7	840905	S	143.0	B	5	NA	SH	INSTRU	Startup testing. Load increased per Shift Supervisor instructions.
84-8	840911	F	4.22	A	5	NA	CB	PUMPXX	Trip of both recirc pumps. Load reduction.
84-9	840911	S	15.62	B	5	NA	SH	INSTRU	Power derating for startup testing. Load reduction.
84-10	840911	S	22.28	B	5	NA	CB	PUMPXX	Testing of scoop tubes on recirc pumps.

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance of Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Continuations
5-Load Reduction
9-Other (Explain)

4
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit I - Same Source

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UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH September

DOCKET NO. 50-366
UNIT NAME Hatch 2
DATE 10-4-84
COMPLETED BY M. McBay
TELEPHONE 912-367-7851

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report #	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
84-11	840912	S	57.85	F	5	NA	RC	CONROD	Pulling rods per STA instructions for ramp up.
84-12	840915	F	117.83	A	5	NA	HH	DEMINX	Load reduction because of condensate demin problems.
84-13	840921	F	37.77	A	3	NA	CD	VALVEX	Rx scram from inboard MSIV's drifting closed.
84-14	840923	S	17.88	F	3	NA	SH	INSTRU	Startup from above rx scram.
84-15	840924	F	4.73	A	5	NA	HH	PUMPXX	Condensate booster pump tripped. Load reduction.
84-16	840926	F	17.58	A	5	NA	CB	PUMPXX	"B" recirc pump drifting down. Load reduction.
84-17	840930	F	12.25	A	3	NA	HC	FILTER	Rx scram on loss of condenser vacuum.

1
F: Forced
S: Scheduled

2
Reason:
A-Equipment Failure (Explain)
B-Maintenance of Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

3
Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Continuations
5-Load Reduction
9-Other (Explain)

4
Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

5
Exhibit I - Same Source

(9/77)

Georgia Power Company
Post Office Box 439
Baxley, Georgia 31513
Telephone 912 367-7781
912 537-9444

Georgia Power

Edwin I. Hatch Nuclear Plant

October 5, 1984
GM-84-889

PLANT E. I. HATCH
NRC Monthly Operating Report

Office of Plans and Schedules
Directorate of Licensing
United States Nuclear Regulatory Commission
Washington, D. C. 20545

Dear Sir:

Per Tech Specs section 6.9.1.6 please find attached the NRC Monthly Operating Report for Hatch Unit 1, Docket #50-321, and for Hatch Unit 2, Docket #50-366.

H. C. Nix
General Manager

HCN/CTJ/JAB/hh

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