

UNITED STATES NUCLEAR REGULATORY COMMISSION
IN THE MATTER OF NORTHERN STATES POWER COMPANY

DOCKET NOS. 50-282, 50-306

PRAIRIE ISLAND NUCLEAR GENERATING PLANT

RECEIPT OF ADDENDUM TO PETITION FOR DIRECTOR'S DECISION UNDER 10 CFR 2.206

Notice is hereby given that by letter dated March 13, 1996, the Nuclear Information and Resource Service and the Prairie Island Coalition request that the U.S. Nuclear Regulatory Commission (NRC) take immediate action with regard to steam generator tube inspections at the Prairie Island Nuclear Generating Plant. The letter was a second addendum to an earlier Petition dated June 5, 1995. A first addendum was received from the Petitioners in a letter dated February 19, 1996.

The Petitioners request that the NRC require Northern States Power Company to place Prairie Island Nuclear Generating Plant Units 1 and 2 in mid-cycle outages to inspect the full length of the steam generator tubes using the Zetec Plus Point Probe and any state-of-the art eddy current test capable of finding circumferentially oriented cracking. If these inspections are not performed, the Petitioners request that the NRC hold an informal public hearing in or near Red Wing, Minnesota, to inform the public why such inspections are not needed.

As the basis for this request, the Petitioners state that the NRC and nuclear utilities operating pressurized-water reactor facilities in the U.S. have known for years that the bobbin coil eddy current inspection probe is not effective at finding circumferentially oriented cracking. Circumferential and axial cracks have been found in the tubesheet region of steam generators; inspections with advanced technology probes that are effective in detecting

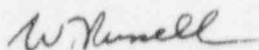
circumferential cracks, such as the Plus Point Probe, are performed in the tubesheet region. The Petitioners state that since axially oriented cracks have been identified in steam generator free-span regions, circumferentially oriented cracks may also be present there. Thus a full-length tube inspection using state-of-the art eddy current probe technology would be prudent.

This addendum to the Petition is being treated pursuant to 10 CFR 2.206 of the Commission's regulations and has been referred to the Director of the Office of Nuclear Reactor Regulation. As provided by 10 CFR 2.206, appropriate action will be taken on the Petition within a reasonable time. By letter dated April 22, 1996, the Director denied the request for immediate action to require that Prairie Island Units 1 and 2 be placed in mid-cycle outages to inspect the full length of the steam generator tubes.

Copies of the addenda to the Petition and the Director's letter are available for inspection at the Commission's Public Document Room at 2120 L Street, NW., Washington, D.C., and at the Local Public Document Room, Minneapolis Public Library, Technology and Science Department, 300 Nicollet Mall, Minneapolis, Minnesota 55401.

Dated at Rockville, Maryland, this 22nd day of April 1996.

FOR THE NUCLEAR REGULATORY COMMISSION



William T. Russell, Director
Office of Nuclear Reactor Regulation

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FOR THE NUCLEAR REGULATORY COMMISSION

Original Signed By:
William T. Russell, Director WILLIAM T. RUSSELL
Office of Nuclear Reactor Regulation

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