

Public Service Electric and Gas Company P.O. Box 236 Hancocks Bridge, New Jersey 08038-0236

Nuclear Business Unit

APR 2 2 1996

LR-N96103

U. S. Nuclear Regulatory Commission Document Control Desk Washington, DC 20555

Attn.: Document Control Desk

Dear Sir:

HOPE CREEK GENERATING STATION LICENSE NO. NPF-57 DOCKET NO. 50-354 UNIT NO. 1 LICENSEE EVENT REPORT NO. 95-033-04

This Licensee Event Report entitled "Technical Specification Surveillance Requirement Implementation Deficiencies" is being submitted pursuant to the requirements of 10CFR50.73(a)(2)(i)(B).

This supplement documents a discovery by the Technical Specification Surveillance Improvement Program (TSSIP). As stated in LER 95-033-01, additional Technical Specification surveillance implementation deficiencies discovered by TSSIP with minimal safety significance will be documented in supplements to this LER on a periodic basis until completion of the TSSIP project.

Sincerely,

marks/11

M/. E./Reddemann General Manager -Hope Creek Operations

Attachment LER SORC Mtg. 96-046 JJK

C Distribution LER File 3.7

9604290082 960422 PDR ADOCK 05000354 S PDR The power is in your hands.

NRC FORM 366 (4-95)

U.S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT (LER)

(See reverse for required number of digits/characters for each block)

APPROVED BY OMB NO. 3150-0104 EXPIRES 04/30/98

EXTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS MANDATORY INFORMATION COLLECTION REQUEST: 50.0 HRS. REPORTED LESSONS LEARNED ARE INCORPORATED INTO THE LICENSING PROCESS AND FED BACK TO INDUSTRY. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (T-6 F33), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

DOCKET NUMBER (2)

05000354

PAGE (3)

1 OF 11

FACILITY NAME (1)

Hope Creek Generating Station

TITLE (4)

Technical Specification Surveillance Requirement Implementation Deficiencies

EVEN	NT DATI	E (5)		LER	NUMBE	R (6	3)	REPOR	RT DAT	E(7)		OTHER FAC	ILITIES INVOLVED (8)
MONTH	DAY	YEAR	YEAR		SEQUENTIAL NUMBER		REVISION	MONTH	DAY	YEAR	FACILITY	YNAME	DOCKET NUMBER
11	14	95	95		033		04	4	22	96	FACILITY	NAME	DOCKET NUMBER 05000
OPERA	TING	1	THIS R	EPO	RT IS SI	JBN	ITTED P	URSUAN	TTOT	HE RE	QUIREN	ENTS OF 10 CFR	§: (Check one or more) (11)
MODE	E (9)		20.	2201	(b)			20.2203	(a)(2)(v)	×	50.73(a)(2)(i)(B)	50.73(a)(2)(viii)
POW	ER	100	20.	2203	(a)(1)			20.2203	(a)(3)(i)	1		50.73(a)(2)(ii)	50.73(a)(2)(x)
LEVEL	. (10)	100	20.	2203	(a)(2)(i)			20.2203	(a)(3)(ii)		50.73(a)(2)(iii)	73.71
	CARGES AVENUES	human da ana ana ana ana ana ana ana ana an	20.	2203	(a)(2)(ii)	-		20.2203	(a)(4)	Carlos Calendar	×	50.73(a)(2)(iv)	OTHER
			the second s		(a)(2)(iii			50.36(c)	CARD COLOR OF THE OWNER OF			50.73(a)(2)(v)	Specify in Abstract belo or in NRC Form 366A
		20.2203(a)(2)(iv)				50.36(c)(2)				50.73(a)(2)(vii)	OF IN NIKE FORM 306A		

NAME

LICENSEE CONTACT FOR THIS LER (12)

TELEPHONE NUMBER (Include Area Code)

(609) 339-5429

Jeff Keenan, Licensing and Regulation

and the second		COMPLETE O	NE LINE FOR E	ACH COMPO	NENT F	AILURE DE	SCRIBED I	N THIS REPO	RT (13)		
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACT	URER	REPORTABLE TO NPROS
D	cc	V	L200	Y							
YES (If yes,			L REPORT EXPE	CTED (14)	X NO		SUBN	ECTED MISSION 'E (15)	MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)

LER 95-033-00 described two events that occurred due to identification of a Technical Specification (TS) surveillance test inadequacy. On 11/14/95, the Technical Specification Surveillance Improvement Program (TSSIP) team determined that the undervoltage auxiliary relays were not adequately tested in accordance with the LOGIC SYSTEM FUNCTIONAL TEST requirements. As stated in LER 95-033-02, supplements would be transmitted to document additional findings of the TSSIP team. In LER 95-033-03, the TSSIP team determined that High Pressure Coolant Injection System (HPCI) valves have not been periodically verified to automatically actuate in response to a high suppression chamber water level in accordance with TS requirements. On 3/25/96, the TSSIP team determined that certain primary containment penetration test and drain valves were not periodically verified to be closed in accordance with the requirements of TS section 4.6.1.1.b. This supplement provides the details of that event, which is being reported under the provisions of 10CFR50.73(a)(2)(i)(B). These conditions have existed due to ineffective reviews of procedures that implement TS requirements. Corrective actions include a comprehensive review of procedures implementing TS surveillance requirements and

surveillance test procedure revisions.

NRC FORM 366A (4-95)		U.S	NUCLEAP R	EGULATO	DRY CO	MMISS	SION
(4.95) LICENSEE EVENT REPORT (LER) TEXT CONTINUATION FACILITY NAME (1) Hope Creek Generating Station TEXT (If more space is required, use additional copies of NRC Form 366A) (17) PLANT AND SYSTEM IDENTIFICATION							
TEXT CONTINUATION FACILITY NAME (1) DOCKET NUMBER (2) LER NUMBER (6) PAGE YEAR SEQUENTIAL REVISION Hope Creek Generating Station 05000354 95 033 04 2 O TEXT (If more space is required, use additional copies of NRC Form 366A) (17)		AGE (3)				
		YEAR	SEQUENTIAL NUMBER	REVISION			
Hope Creek Generating Station	05000354	95	033	04	2	OF	11
FEXT (If more space is required, use additional copies of NR	C Form 366A) (17)	houseneers			the spinster connect	and the products	water a
PLANT AND SYSTEM IDENTIFICATION							
General Electric - Boiling Water	Reactor (BWR/4)						
Safety Auxiliaries Cooling Syste Reactor Water Cleanup System (RW 4.16 KVAC - EIIS Identifier {EB} Emergency Diesel Generator - EIIS High Pressure Coolant Injection Sy	<i>ICU) -</i> EIIS Identi Identifier {EK}	fier	{CE}	20}			
IDENTIFICATION OF OCCURRENCE							
Discovery dates: 11/14/95, 12 ESF actuation date: 11/16/95 Problem Reports: 951114174, 9 and 960322230	51116123, 9512121					5156	
CONDITIONS PRIOR TO OCCURRENCE							
For the events in this LER the p	lant was in vario	us o	peratior	nal co	ndit	ion	s.
DESCRIPTION OF OCCURRENCE							
LER 95-033-00 described two even Technical Specification (TS) Sur rewrites the original LER to des surveillance implementation defi Specification Surveillance Improve	veillance Test ina cribe an additiona ciency identified	adeq al o dur	uacy. I ccurrenc ing the	his s e of	uppl a TS	emei	
On November 14, 1995, during the 'Cooling System Actuation Instrument undervoltage auxiliary relays were the LOGIC SYSTEM FUNCTIONAL TEST result, the vital bus undervoltage Action Statement was entered for the surveillance testing.	ntation", it was de e not adequately to (LSFT) requirements e relays were decla	eter este s of ared	mined th d in acc TS 4.3. inopera	at the ordance 3.2. ble, a	e ce w As arid	ith a	
The surveillance test was revised identified. On November 16, 1995 surveillance on the 'A' 4 kV vita	, during the perfor	rman	ce of th	e revi			0

surveillance on the 'A' 4 kV vital bus, a bus transfer occurred at 0521. The 'A' Loss of Offsite Power (LOP) Sequencer initiated per plant design. A four-hour report was made to the NRC at 0841 in accordance with 10CFR50.72(b)(2)(ii).

On December 12, 1995, the TSSIP team determined that channel calibrations for the Reactor Water Cleanup System (RWCU) instrumentation, required by TS Table 3.3.2-1, were not being performed appropriately. Specifically, the

NRC FORM 366A (4-95)	ng nan ang ang ang ang ang ang ang ang a	U.S	NUCLEAR RE	GULATO	RYC	OMMIS	SION
	SEE EVENT REPORT (LE TEXT CONTINUATION	R)					
FACILITY NAME (1)	DOCKET NUMBER (2)		LER NUMBER	(6)	1	AGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
Hope Creek Generating Station	05000354	95	033	04	3	OF	11

DESCRIPTION OF OCCURRENCE (Continued)

RWCU ambient temperature instrumentation and differential temperature instrumentation channel calibrations have not included a sensor calibration as specified in TS Definition 1.4, CHANNEL CALIBRATION.

The RWCU instrumentation was not required to be operable at the time of discovery of the deficient surveillances and no TS Actions were required to be taken. However, this condition has existed since plant startup and TS Actions were not previously implemented as required by Table 3.3.2-1. Therefore, this condition is being reported under the provisions of 10CFR50.73(a)(2)(i)(B).

On January 4, 1996, the TSSIP team determined that the SACS heat exchanger inlet valves EG-HV-2491 A&B and EG-HV-2494 A&B have not been tested in accordance with the requirements of TS surveillance requirement 4.7.1.1.b.1. This surveillance requirement specifies that at leas' once per 18 months, during shutdown, these valves actuate to their correct position on the appropriate test signal (i.e., a SACS pump start signal).

At 1719 hours on January 4, 1996, the SACS heat exchanger inlet valves were declared inoperable and administratively controlled to ensure performance of the valves safety function.

On February 26, 1996, the TSSIP team determined that several HPCI system valves have not been periodically tested in accordance with TS surveillance requirement 4.5.1.c.2.b. This surveillance requirement states that, "At least once per 18 months, verify that the suction is automatically transferred from the condensate storage tank to the suppression chamber on a condensate storage tank water level-low signal and on a suppression chamber-water level high signal.'' Specifically TSSIP determined that: 1) the HPCI system suppression pool suction valve (BJ-HV-F042) has not been verified to open on a suppression chamber-water level high signal; 2) the HPCI system condensate storage tank (CST) suction valve (BJ-HV-F004) has not been verified to close on a suppression chamber-water level high signal; and 3) the HPCI full flow test line valve (BJ-HV-F011) h s not been verified to close on a suppression chamber-water level high sig 1. Since Hope Creek was in an Operational Condition where HPCI was not required to be operable, administrative controls were used to ensure the valves were properly tested in accordance with the TS requirement: .

On March 25, 1996, the TSSIP team determined that certain primary containment penetration test and drain valves were not periodically verified to be closed in accordance with the requirements of TS 4.6.1.1.b. This surveillance requirement states that, 'At least once per 31 days (verify) that all primary containment penetrations not capable of being closed by OPERABLE containment automatic isolation valves and required to be closed during accident conditions are closed by valves...''.

NRC FORM 366A (4-96)	Ann an the for the second s	U.S	. NUCLEAR RI	GULATO	RY COMMISSION
	LICENSEE EVENT REPORT (LE TEXT CONTINUATION	R)			
FACILITY NAME (1)	DOCKET NUMBER (2)		LER NUMBER	(6)	PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION	
Hope Creek Generating Statio	n 05000354	95	033	04	4 OF 11

DESCRIPTION OF OCCURRENCE (Continued)

Specifically, TSSIP determined that several test and drain valves were omitted from the procedure that verifies primary containment integrity for TS 4.6.1.1.b. The valves were verified to be in their proper closed position and no additional TS actions were warranted. The procedure that verifies primary containment integrity for TS 4.6.1.1.b was revised to incorporate the excluded valves. A review of all primary containment penetrations is on-going to ensure all appropriate TS 4.6.1.1.b components are identified.

ANALYSIS OF OCCURRENCE

As a Corrective Action from LER 95-017, a Technical Specification Surveillance Improvement Program (TSSIP) had been initiated. The charter of this project is to compare the TS surveillance requirements (with the exception of the Technical Specification 4.0.5 requirements) to the established surveillance procedures to verify that all requirements are met.

During TSSIP review of TS 3.3.3, "Emergency Core Cooling System Actuation Instrumentation", it was determined that individual contacts, and their configuration, from the undervoltage auxiliary relays and the degraded voltage relays were not tested in accordance with the LSFT requirements of TS 4.3.3.2. These contacts are for the load shedding of major 4.16 kV loads of the vital bus, incoming feeder breaker trips and lock outs, diesel generator start permits, and input to the load sequencer. The LSFT is required to be performed at least once per 18 months.

On November 15, 1995 both the degraded voltage and the bus undervoltage surveillance procedures were revised to incorporate the contacts and wiring that needed to be tested to satisfy the TS surveillance testing.

While testing the 'A' Vital Bus (10A401), a bus transfer occurred when the technician inadvertently touched an adjacent terminal. The bus transfer performed as designed. The 'A' Loss of Offsite Power (LOP) Sequencer initiated per plant design. The affected systems performed as expected and testing was terminated.

In December 1995, the TSSIP reviewed the implementing procedures for surveillance requirements associated with the RWCU system. The suction line (reactor coolant pressure boundary portion) of the RWCU system contains two motor operated isolation valves that automatically close in response to, among other signals, RWCU equipment compartment high ambient temperature and high differential temperature across the RWCU equipment compartment ventilation ducts. The event concerned the channel calibrations performed for these signals.

NRC FORM 366A		U.S	. NUCLEAR RE	GULATO	RYC	DMMIS	SION
	LICENSEE EVENT REPORT (LE TEXT CONTINUATION	R)					
FACILITY NAME (1)	DOCKET NUMBER (2)		LER NUMBER	(6)	F	AGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
Hope Creek Generating Statio	n 05000354	95	033	04	5	OF	11

ANALYSIS OF OCCURRENCE (Continued)

In the past, channel calibrations for instrument channels having resistance temperature detector (RTD) or thermocouple (T/C) sensors have been completed by performing an inplace qualitative assessment of sensor behavior and normal calibration of the remaining adjustable devices in the channel. This test methodology is consistent with standard industry practice and has been considered to satisfy the surveillance requirements. However, the TSSIP team determined that these surveillance procedures were inconsistent with the literal requirements specified in TS 1.4, CHANNEL CALIBRATION, which requires calibration of the sensor regardless of whether the channel has an RTD or T/C sensor. Unlike other nuclear plant TS, there is no qualifying TS Table notes in the Hope Creek TS to exempt RTDs and T/Cs from the sensor calibration requirements.

The qualifying note was added to other plant's TS since calibration of RTDs and T/Cs cannot usually be performed in place. Removal and subsequent reinstallation of the sensors introduces a potential for an undetectable failure and alarm considerations that outweighs the benefits of the sensor calibration. In lieu of sensor calibration, an inplace qualitative assessment of sensor behavior is performed. This position was adopted in NUREG-1433, ''Improved Standard Technical Specifications for General Electric BWR/4 Plants.''

Failure to appropriately perform the surveillances for the RWCU instrumentation requires entry into the TS Action Statement specified in Table 3.3.2-1. Since this did not occur, this event is reportable under the provisions of 10CFR50.73(a)(2)(i)(B).

Additional review performed by the TSSIP identified that this condition exists for all of the RTD and T/C sensors for instrumentation listed in TS Table 4.3.2.1-1, Isolation Actuation Instrumentation Surveillance Requirements, Table 4.3.7.4-1, Remote Shutdown Monitoring Instrumentation Surveillance Requirements and Table 4.3.7.5-1, Accident Monitoring Instrumentation Surveillance Requirements.

In January 1996, the TSSIP team determined that TS surveillance requirement 4.7.1.1.b.1 has not been performed for the SACS heat exchanger inlet valves. The SACS is designed to provide cooling water to the engineered safety feature equipment, including the residual heat removal heat exchangers, during normal operation, normal plant shutdown, loss of offsite power and loss of coolant accident conditions. Failure to demonstrate that the SACS heat exchanger inlet valve actuates to the open position upon its associated pump start signal at the specified TS frequency and Operational Condition requires entry into the SACS Action Statement for LCO 3.7.1.1, ''with both SACS subsystems inoperable, immediately initiate measures to place the unit in at least Hot Shutdown within the next 12 hours.'' Since this did not occur, this event is reportable under the provisions of 10CFR50.73(a)(2)(i)(B).

NRC FORM 366A (4-95)

NRC FORM 366A (4-95)	e na en entre de celle : entre de celle en de la sense :	U.S	. NUCLEAR RE	GULATO	RY COMMISSION
	LICENSEE EVENT REPORT (LE TEXT CONTINUATION	R)			
FACILITY NAME (1)	DOCKET NUMBER (2)		LER NUMBER	(6)	PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION	
Hope Creek Generating Statio	n 05000354	95	033	04	6 OF 11

ANALYSIS OF OCCURRENCE (Continued)

On February 26, 1996, TSSIP determined that TS surveillance requirement 4.5.1.c.2.b had not been performed for several HPCI valves. Failure of the surveillance test procedures to require verification of the automatic alignment of the subject HPCI valves has existed since initial plant startup. HPCI is designed to provide make-up during a small break Loss of Coolant Accident (LOCA). HPCI may be used for reactor vessel inventory or pressure control whenever the reactor vessel is pressurized and isolated from the feedwater and/or main steam system. The HPCI pump normally draws water from the CST and discharges to the core spray and feedwater system piping. A full flow test line (back to the CST) is provided on the HPCI pump discharge line to allow testing of the system during normal plant operations without injecting water into the reactor vessel.

Surveillance test procedures have not required verification of the automatic actuation capability of the subject HPCI valves. Failure to perform these surveillances in accordance with the frequency specified in the TS requires actions to be taken to enter at least Hot Shutdown within 12 hours after the allowed outage time expires. Since these actions were not taken, a condition prohibited by the TS occurred, which is reportable under the provisions of 50.73(a)(2)(i)(B).

On March 26, 1996, TSSIP determined that TS surveillance requirement 4.6.1.1.b had not been performed for several primary containment penetration test and drain valves. Failure of the surveillance test procedure to verify all primary containment penetration valves has existed since initial plant start-up. The test and drain valves in question are closed valves, one inch or less in diameter, with the downstream piping capped closed. Since the surveillance test procedures have not required verification of all the primary containment penetration valves, the missed TS surveillance is reportable under the provisions 10 CFR 50.73 (a) (2) (i) (B).

NRC FORM 366A (4-95)			. NUCLEAR RI	EGULATO	RY COMMISSI	ION
	LICENSEE EVENT REPORT (LE TEXT CONTINUATION	R)				
FACILITY NAME (1)	DOCKET NUMBER (2)	T	LER NUMBER	(6)	PAGE (3)	ALL
		YEAR	SEQUENTIAL NUMBER	REVISION		
Hone Creek Generating Statig	05000354	OF	022	04	7 OF	11

APPARENT CAUSE OF OCCURRENCE

The cause for the failure to properly test the undervoltage relays is procedural inadequacies due to lack of knowledge of what constitutes a satisfactory LSFT. A contributing factor is the lack of guidance regarding the requirements of LSFTs.

The cause of the bus transfer was a test lead coming into contact with a terminal while the technician was attaching test equipment to a relay. Contributing factors were the decision to perform the test while the bus was energized and inadequate job planning in that the effects of conducting the test in an energized cubicle that was not designed for test leads were not completely analyzed.

The apparent cause of the RTD and T/C deficient channel calibrations was the ineffective review of surveillance procedures intended to satisfy Hope Creek's TSs during the near-term operating license stage in the 1980s. A contributing factor to this issue was plant staff habit intrusion. Hope Creek was performing the RTD and T/C sensor calibrations in accordance with current industry practice and it was assumed that the intent of the TS was being met. The same apparent cause is attributed to the deficient surveillances performed on the other Isolation Actuation, Remote Shutdown and Accident Monitoring Instrumentation.

The apparent cause of the missed surveillance tests of the SACS heat exchanger isolation valves is the same as that for the RTD and T/C deficient channel calibrations: ineffective procedures/inadequate review of surveillance activities intended to satisfy Hope Creek's TS during the near-term operating license stage in the 1980s.

The apparent cause of the missed surveillance tests of the subject HPCI valves is the same as that for the SACS heat exchanger isolation valves: inadequate review and approval of surveillance activities intended to satisfy Hope Creek's TS during the near-term operating license stage in the 1980s.

The apparent cause of the missed surveillance tests to verify all the primary containment penetration test and drain valves is inadequate review and approval of surveillance activities intended to satisfy TS.

NRC FORM 366A (4-95)		U.S	NUCLEAR RE	GULATO	RYCO	OMMIS	SION
	LICENSEE EVENT REPORT (LE TEXT CONTINUATION	R)					
FACILITY NAME (1)	DOCKET NUMBER (2)		LER NUMBER	(6)	P	AGE (3)
an a substitution of the		YEAR	SEQUENTIAL	REVISION NUMBER			
Hope Creek Generating Static	n 05000354	95	033	04	8	OF	11

SAFETY SIGNIFICANCE

Undervoltage relay testing:

Although the undervoltage and degraded voltage relays were declared inoperable due to nonperformance of a surveillance requirement, reasonable assurance existed that the Emergency Diesel Generators would start and energize the bus on a loss of power coincident with a Loss of Cooling Accident, and that all required ESF loads would sequence on the vital bus. This assurance is based on previous successful past performances of the integrated Emergency Diesel Generator test. Additionally, performance of testing on the 'A' and 'C' vital busses demonstrated compliance with the LSFT requirements, and showed all required relays and contacts to be operational.

ESF actuation:

Due to the risks associated with the performance of this surveillance test (i.e., loss of the bus), Operations evaluated each load on the associated bus and provided recommendations regarding the use of redundant equipment to minimize the impact to plant operations. Therefore, the safety significance associated with this event was minimal.

RTD and T/C channel calibrations:

Performance of inplace qualitative assessments of RTD and T/C sensor behavior in lieu of sensor calibrations has been determined to be an acceptable method for demonstrating the operability of the isolation function. This method has been accepted by the NRC and described in NUREG 1433 for this instrumentation. Therefore, there is no safety significance of the failure to perform sensor calibrations as specified in the existing TS Definition 1.4 for the RTD and T/C sensors.

SACS heat exchanger inlet valve surveillances:

There was minimal safety significance for the inadequate SACS heat exchanger inlet valve surveillance test procedures. The basis for this minimal impact is: 1) the normal position of the heat exchanger inlet valves is open; 2) the SACS operating procedure directs the operator to verify that the valve opens following a pump start; 3) the valves fail asis, which ensures a suction flow path for pumps previously inservice in the event of a design basis accident; and 4) indications available in the control room make the operator aware of a logic malfunction (causing the valve to not open as required), such that compensatory actions can be initiated.

NRC FORM 366A - (4-95)		U.8	. NUCLEAR RE	GULATO	RY COMMISSION
	LICENSEE EVENT REPORT (LE TEXT CONTINUATION	R)			
FACILITY NAME (1)	DOCKET NUMBER (2)	1	LER NUMBER	(6)	PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION	
Hope Creek Generating Station	05000354	95	033	04	9 OF 11

HPCI valve surveillances:

The normal positions for the subject HPCI valves enable HPCI to function upon an initiation signal without these valves changing position. The position of these valves is verified twice daily. The capability for the HPCI system to automatically take suction from the suppression chamber on a suppression chamber-water level high signal has also been demonstrated within the past 18 months. LERs 95-014-00 and 95-020-01 were written to document two ESF actuations where the HPCI suction realigned to the suppression chamber from the CST on a suppression chamber-water level high signal.

In addition, surveillance testing satisfying the requirements of TS 4.5.1.c.2.b has been completed and demonstrated the capability of the subject valves to automatically actuate on a suppression chamber-water level high signal. Since the operability of the HPCI system was not affected with the subject valves in an off-normal position, there were no adverse safety consequences associated with this event.

Primary Containment penetration test and drain valves:

The normal position for the subject primary containment penetration test and drain valves is the closed position with the downstream piping isolated closed with a secured pipe cap. Positioning of plant components, including valves, is controlled by various administrative means. In this case, the test and drain valves are used as part of leak rate surveillance testing. At the completion of leak rate surveillance testing all repositioned valves are confirmed and independently verified to be left in their proper position. Because this testing is generally the only function of these valves, it is extremely unlikely that the valves would be mispositioned. All the valves have been field verified to be in the correct closed position. Since the valves were verified to be in the correct positions and administrative means were in place to control valve positioning, a past valve mispositioning error is unlikely. Therefore, the safety significance of this event is minimal.

PREVIOUS OCCURRENCES

Failure to follow TS surveillance requirements has been documented in LERs 95-003-00 and supplements, 95-017-00, 95-034-00 and 95-035-00. LER 95-03-00 documented an event where operators performed a surveillance in an operational condition other than that specified by the TS, LER 95-017-00 documented an event where the emergency bus undervoltage logic circuitry was improperly tested, LER 95-034-00 documented a failure to perform Rod Sequence Control System surveillances when required and LER 95-035-00 documented the failure to perform Reactor Mode Switch, Source Range Monitor and Suppression Chamber Level surveillances properly.

NRC FORM 366A (4-95)		U.S	. NUCLEAR RE	EGULATOR	Y COMMISSION
	LICENSEE EVENT REPORT (LE TEXT CONTINUATION	R)			3.3.7.3
FACILITY NAME (1)	DOCKET NUMBER (2)		LER NUMBER	(6)	PAGE (3)
		YEAR	SEQUENTIAL	REVISION	

05000354

10 OF 11

95 -- 033 -- 04

Hope Creek Generating Station

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

PREVIOUS OCCURRENCES (Continued)

In response to LER 95-017-00, the General Manger - Hope Creek Operations chartered the TSSIP to investigate, define, and resolve weaknesses in the TS Surveillance Program. The events described in this LER were identified as a result of the TSSIP.

CORFECTIVE ACTIONS

The implementing procedures for testing the bus undervoltage auxiliary contacts have been revised to defeat the undervoltage trip function during the performance of the test. The TSSIP group independently reviewed the procedures to ensure satisfactory compliance. This was completed prior to performance of the test procedures.

Logic System Functional Testing was performed on the 'B' and 'D' vital busses to demonstrate operability of the undervoltage and degraded voltage relays to satisfy requirements of Surveillance Requirement 4.3.3.1.

The TSSIP review will continue, with particular attention to the Logic System Functional Test Requirements in the other instrumentation specifications. The TSSIP will be completed by December 31, 1996.

The Technical Specification Matrix will be updated to reflect new procedures to comply with the LSFT requirement. This will be performed as the TSSIP identifies issues and will be completed by December 31, 1996.

Position papers were prepared to outline the proper test methodology and acceptance criteria for performance of technical specification surveillances, such as LSFT and Channel Functional Test requirements.

Training based on the site approved position papers will be prepared and incorporated into initial and continuing training programs for personnel responsible for the preparation, review, and approval of logic system surveillance procedures. The initial training will be conducted for licensed operators, system managers, procedure writers, and Station Qualified Reviewers, and will be completed by December 31, 1996.

Guidance was provided to the relay and controls technicians regarding the selection and use of M&TE (specifically M&TE with alligator clips).

The Controls Pre-Job Brief Checklist has been revised to ensure the proper use of M&TE.

	EVENT REPORT (LE		S. NL	JCLEA	RRE	GULATO	DRY C	OMMIS	SION
FACILITY NAME (1)	DOCKET NUMBER (2)		LER	NUME	BER	(6)	F	AGE (3)
		YEAR	-	NUMBER		REVISION			
Hope Creek Generating Station	05000354	95		033		04	11	OF	11
TEXT (If more space is required, use additional copies of NRC	Form 366A) (17)	harmonic scores					Assessment	anundina tora	Avenue.co
CORRECTIVE ACTIONS (Continued)									
The procedures used to conduct the specify the specific alligator cli completed prior to the next time t	lp to be used. Th	is :	rev	visio					
A design change to install test po implemented by the end of the next			ubi	cle:	s W	ill Ł	e		
The TS definition of CHANNEL CALL Operational Condition 3 following inplace qualitative assessments of	g the sixth refue	ling	jõ	utag					þ
The SACS heat exchanger inlet value to ensure performance of the value appropriately teaced to satisfy t	ves' safety funct	ion.		Thes	se	valve	s w	olle ere	ad
Permanent procedure revisions to accordance with the requirements to the performance of the next so	of TS 4.7.1.1.b.	1 wi	.11	be	CO				or
The HPCI surveillance test proced the subject HPCI valves and ensur				o ap	pr	opria	tel	y te	st
The subject HPCI valves have been TS 4.5.1.c.2.b have been satisfie		and	l t	he r	eq	uirem	ent	s of	
The primary containment penetrati surveillance procedure that verif			lv	es w	er	e add	ed 1	to t	he
A review of all primary containme appropriate TS 4.6.1.1.b componen completed by August 31, 1996.									1