NRC Form (9-83)													NUCLEAR REGULATORY COMMISSION APPROVED OMB NO. 3150-0104 EXPIRES: 8/31/86					
FACILITY NAME (1) DOCKET														ET NUMBER (2) PAGE (
	Po	int	B	each	Unit	2						0 5	1010	101	131011	1 OF	012	
TITLE (4	Ir	nadv	er	tent	Actua	ation c	of th	ne Re	acto	r Pro	otectio	_						
	by	r Re	mo	val c	of In:	strumer	it Fi	ises										
EVE	NT DATE	(5)		LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVO				LVED (8)			
MONTH	DAY	YEAR	YE	YEAR SEQUENTIAL NUMBER			MONTH	DAY	DAY YEAR	FACILITY NAMES			DOCKET NUMBER(S)					
FIR										None				0 5 0 0 0			11	
019	2 8	8 4	8	4	ol ol	010	110	2 4	8 4	None	e			0	5 0 10	101		
-		-	THE	REPORT	IS SUBMITTI	ED PURSUANT	O THE R	EQUIREME	NTS OF 10	CFR 8: 10	heck one or more	of the fo	illowing) (1	_			,	
OPERATING MODE (9)				20.402(b)			20.406(c)			X 80.73(a)(2)(iv)			73.71(b)					
POWE				20.405(a)(1)(i)			50.38(c)(1)			50,73(a)(2)(v)			73.71(e)					
(10)	101	010	20.406(a)(1)(B)				50.36(c)(2)			50,73(a)(2)(vii)			OTHER (Specify in Abstract					
				20.405(a)	(1)(iii)		50.73(e)(2)(i)			50.73(a)(2)(viii)	(A)			366A)	n raxt, nem	. rom	
				20,405(a)(1)(iv)			80,73(a)(2)(ii)			50.73(a)(2)(viii)(B)								
				20.408(a)(1)(v)				50.73(a)(2)(iii)			50.73(e)(2)(x)							
						- 1	ICENSEE	CONTACT	FOR THIS	LER (12)								
NAME											TELEPHONE NUMBER							
C. W. Fay, Vice Presiden							ent-N	Nucle	ar P	ower			11 14	2	7,7,-	1218	1111	
					COMPLETE	ONE LINE FOR	EACH CO	OMPONENT	FAILURE	DESCRIBE	D IN THIS REPO	RT (13)						
CAUSE	SYSTEM	СОМР	ONEN	MANUFAC- TURER		REPORTABLE TO NPRDS			CAUSE	SYSTEM	COMPONENT	MANUFAC- TURER		REPORTABLE TO NPROS				
	1	1			11						111		1.1					
													-	T				
		-1	1									1	11					
					SUPPLEM	ENTAL REPORT	EXPECT	ED (14)				_	EXPECT	ED	MONTH	DAY	YEAR	
YES (If yes, complete EXPECTED SUBMISSION DATE)												SUBMISSION DATE (18)						
						single space tun	12	7						-				

While performing hot rod drop testing on a previously shut down reactor, an inadvertent reactor protection system actuation (trip) occurred when the instrument fuses for source range Channel 31 were removed without bypassing the trip function. It has been determined that the cause of this trip was personnel error due to a failure to follow the appropriate procedure.

8411130191 841024 PDR ADOCK 05000301 S PDR LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

FACILITY NAME (1)

Point Beach Unit 2

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

Page (3)

YEAR SEQUENTIAL REVISION
NUMBER NUMBER NUMBER

0 |5 |0 |0 |0 |3 |0 |1 |8 |4

010 012 OF 012

01014

TEXT (If more space is required, use additional NRC Form 366A's) (17)

During hot rod drop testing with two-thirds of the rods out, source range Channel 31 failed to energize when the "automatic unblocking" signal was received. An attempt to manually energize the detector, using the intermediate range defeat pushbuttons, also proved unsuccessful. The I&C Supervisor was notified and further investigation revealed that the detector had energized but no counts were indicated. The detector voltage then began to drop and the source range loss of detector voltage alarm was again received. The Duty Operating Supervisor then removed the instrument fuses to remove detector power. The removal of the instrument fuses without bypassing the trip function caused an actuation of the reactor protection system. After appropriate emergency procedures were completed in response to the actuation, Channel 31 was properly removed from service in accordance with the procedure. Since plant Technical Specifications only require one source range channel to be operable, no violation of the Technical Specification Limiting Condition for Operation occurred.

A discussion with the Duty Operating Supervisor indicated that he realized his mistake immediately. A review of the applicable procedure was also conducted and it was concluded that the procedure is adequate. This event is being discussed with all operators in order to prevent future occurrences. No further action is considered to be required.



October 24, 1984

Mr. J. G. Keppler, Regional Administrator Office of Inspection and Enforcement, Region III U. S. NUCLEAR REGULATORY COMMISSION 799 Roosevelt Road Glen Ellyn, Illinois 60137

Dear Mr. Keppler:

LICENSEE EVENT REPORT NO. 84-004-00

INADVERTENT ACTUATION OF THE REACTOR PROTECTION SYSTEM

POINT BEACH NUCLEAR PLANT, UNIT 2

Enclosed is Licensee Event Report No. 84-004-00 which provides a description of an inadvertent actuation of the Unit 2 reactor protection system initiated by removal of instrument fuses from source range Channel 31 while in a shutdown condition reportable in accordance with 10 CFR 50.73(a)(2)(iv), "Any event or condition that resulted in manual or automatic actuation of any engineered safety feature, including the reactor protection system".

Very truly yours,

au fax

Vice President-Nuclear Power

C. W. Fay

Enclosure

Copy to NRC Resident Inspector

OCT 2 6 1984

IE22