



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 80 TO FACILITY OPERATING LICENSE NO. DPR-66

DUQUESNE LIGHT COMPANY

OHIO EDISON COMPANY

PENNSYLVANIA POWER COMPANY

BEAVER VALLEY POWER STATION, UNIT NO. 1

DOCKET NO. 50-334

Introduction

By letter dated May 21, 1984, Duquesne Light Company (the licensee) requested a number of changes to the Technical Specifications appended to Operating License No. DPR-66 for Beaver Valley Unit 1. The staff has completed review of a few of these changes and the results are described as follows.

Evaluation and Discussion

The licensee requested that new technical specifications be imposed on the narrow range containment sump level instrument. In the meantime, the specifications are changed such that either the sump level instrument or sump discharge flow measurement system need be operable during normal operation. The changes are:

1. Adding the narrow range sump level instrument in the limiting condition for operation in Section 3.4.6.1, as per Updated Final Safety Analysis Report (UFSAR) Section 7.3.1.3.1. The narrow range sump level instrument provides an additional method of leakage indication and monitors the sump level under normal operating conditions.
2. Narrow range level instrument operability is added to the Action Statement B in Section 3.4.6.1 as follows:

"With the containment sump discharge flow measurement system and narrow range level instrument inoperable, restore at least one inoperable system to OPERABLE status within seven days or be in at least HOT STANDBY within the next six hours and in COLD SHUTDOWN within the following 30 hours."
3. Adding the Action Statement C in Section 3.4.6.1, to permit the plant to be restarted following a plant trip. Alternate and diverse methods of monitoring reactor coolant system (RCS) leakage are available: containment sump discharge flow measurement or narrow range sump level. Therefore, should one of these systems become inoperable, the other system would be available to monitor and detect RCS leakage.

4. Surveillance requirements for logging the narrow range level indication every 12 hours is included in Section 4.4.6.1.

The above four changes to the technical specifications will have the effect of requiring either the sump level instrument or the sump discharge flow measurement system be operable during normal operation. Previously, only the latter was required. Either is capable of detecting reactor coolant leakage and would provide prompt notification to the operators. All components and systems are already in place and in use; no hardware change is involved with this amendment. We conclude that the requested changes are acceptable.

The licensee also requested a number of changes to the Technical Specifications to correct administrative errors due to previous amendments. These corrections are all acceptable and are issued as amended Pages 3/4 3-56, 3/4 7-14 and 5-1b of the Technical Specifications.

The licensee further requested that a footnote be added to Specification 3.4.1.5, "Isolated Loop Startup". The footnote would specify that the specification is applicable only when fuel is present in the reactor vessel. This specification is intended to prevent a reactivity transient due to the injection of cool water from the startup of an idle loop. Therefore, with no fuel in the vessel, there can be no reactivity transient and no need for this specification. The requested change merely clarifies the specification and is acceptable to the staff.

Environmental Consideration

This amendment involves only changes in administrative procedure and requirements. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR Section 51.22(c)(10). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

Conclusion

We have concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, and (2) such activities will be conducted in compliance with the Commission's regulations and the issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public.

Dated: October 9, 1984

Principal Contributor:

R. Goel



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

October 4, 1984

Docket Nos. 50-348
and 50-364

Mr. R. P. McDonald
Senior Vice President
Alabama Power Company
Post Office Box 2641
Birmingham, Alabama, 35291

Dear Mr. McDonald:

The Commission has issued the enclosed Amendment No. 49 to Facility Operating License No. NPF-2 and Amendment No. 40 to NPF-8 for the Joseph M. Farley Nuclear Plant, Unit Nos. 1 and 2, respectively. The amendments consist of changes to the Technical Specifications in response to your application transmitted by letter dated March 2, 1984.

The amendments modify the Technical Specifications (TS) to upgrade existing TSs for meteorological monitoring instrumentation (TS 3/4.3.3.4) to conform with the Commission Standard Technical Specifications. The change also adds surveillance requirements and minimum number of operable channels for air temperature delta T to Tables 3.3-8 and 4.3-5.

A copy of the related Safety Evaluation is enclosed. A Notice of Issuance will be included in the Commission's next regular monthly Federal Register notice.

Sincerely,

A handwritten signature in cursive script that reads "Edward A. Reeves".

Edward A. Reeves, Project Manager
Operating Reactors Branch #1
Division of Licensing

Enclosures:

1. Amendment No. 49 to NPF-2
2. Amendment No. 40 to NPF-8
3. Safety Evaluation

cc: w/enclosures
See next page

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Mr. R. P. McDonald
Alabama Power Company

Joseph M. Farley Nuclear Plant
Units 1 and 2 -

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