#### OPERATING DATA REPORT

DOCKET NO: 50-368 DATE: October 1984 COMPLETED BY: L.S. Bramlett

TELEPHONE: 501-964-3145

# OPERATING STATUS

Unit Name: Arkansas Nuclear One - Unit 2 Reporting Period: September 1-30, 1984 2. Licensed Thermal Power (MWt): 2815 3. 4. Nameplate Rating (Gross MWe): 942.57 5. Design Electrical Rating (Net MWe): 912

Maximum Dependable Capacity (Gross MWe): 897 6. Maximum Dependable Capacity (Net MWe): 858 7.

8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since

Last Report, Give Reasons:

9. Power Level To Which Restricted. If Any (Net MWe): None

10. Reasons For Restrictions. If Any: None

11	Name is Described Desired	MONTH	YR-TO-DATE	CUMULATVE
11. 12.	Hours in Reporting Period Number of Hours Reactor was	720.0	6,575.0	39,599.0
	Critical	658.8	5,535.7	27,162.8
13.	Reactor Reserve Shutdown			
	Hours	0.0	0.0	1,430.1
14.	Hours Generator On-Line	654.3	5,369.5	26,319.8
15.	Unit Reserve Shutdown Hours	0.0	0.0	75.0
16.	Gross Thermal Energy Generated			
	(MWH)	1,715,685.0	13,839,425.0	66,388,965.0
17.	Gross Electrical Energy			
	Generated (MWH)	570,822.0	4,606,135.0	21,623,086.0
18.	Net Electrical Energy			
	Generated (MWH)	544,498.0	4,394,133.0	20,600,473.0
19.	Unit Service Factor	90.9	81.7	66.5
20.	Unit Availability Factor	90.9	81.7	66.7
21.	Unit Capacity Factor			
	(Using MDC Net)	88.1	77.9	60.6
22.	Unit Capacity Factor			
	(Using DER Net)	82.9	73.3	57.0
23.	Unit Forced Outage Rate	9.1	7.8	18.1
24.	Shutdowns Scheduled Over Next 6 Each): None	Months (Type, Da	ite, and Duration	
25.	If Shut Down At End of Report Pe	riod Estimated	Date of	
	Startun:	rous Escindece	0000	

26. Units in Test Status (Prior to Commercial Operation):

Forecast Achieved INITIAL CRITICALITY INITIAL ELECTRICITY COMMERCIAL OPERATION

8411130057 840930 PDR ADDCK 05000368

### AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO: 50-368

UNIT: Two

DATE: October 1984

L.S. Bramlett

TELEPHONE: 501-964-3145

# INSTRUCTION

On this format, list the average daily unit power level in MWe-Net for each day in reporting month. Compute to the nearest whole megawatt.

# OPERATING SUMMARY SEPTEMBER 1984

UNIT 2

The unit began the month shutdown in order to repair a RCP seal leak and a steam generator manway leak. The unit was placed on line on September 3 and reached 100% full power (FP) on September 7. On September 24, power was reduced to 75% to repair an EH fluid leak on the "B" MFW pump EHC system. The unit was returned to 100% power on September 25 and remained there through September 30, when power was reduced to 75% because of secondary chemistry problems.

## UNIT SHUTDOWNS AND POWER REDUCTIONS REPORT FOR SEPTEMBER 1984

DOCKET NO 50-368 ANO-2 UNIT NAME 10/01/84 DATE L.S. Bramlett COMPLETED BY 501-964-3145 TELEPHONE

No.	Date	Type <sup>1</sup>	Duration (Hours)	Reason <sup>2</sup>	Method of Shutting Down Reactor <sup>3</sup>	Licensee Event Report #	System Code <sup>4</sup>	Component Code <sup>5</sup>	Cause & Corrective Action to Prevent Recurrence
8408	840828	F	65.7	A	4	2-84248	ZZ	ZZZZZZ	The unit tripped due to a dropped CEA. The unit then went to CSD to repair a faulty RCP seal and a leaking steam generator manway.

F: Forced

S: Scheduled

Reason: A-Equipment Failure (Explain) 1-Manual B-Maintenance or Test C-Refueling D-Regulatory Restriction E-Operator Training & License Examination F-Administrative G-Operational Error (Explain)

H-Other (Explain)

Method: 2-Manual Scram. 3-Automatic Scram. 4-Continuation 5-Load Reduction

9-Other

Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

Exhibit 1 - Same Source

DATE: September 1984

# REFUELING INFORMATION

<ol> <li>Scheduled date for next refueling shutdown. April 1985</li> <li>Scheduled date for restart following refueling. July 1985</li> <li>Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? If answer is yes, what, in general, will there be? If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?</li> <li>Yes, some software changes to the Core Protection Calculators will be made, and this will cause some technical specification changes.</li> <li>Scheduled date(s) for submitting proposed licensing action and supporting information. February 1985</li> <li>Important licensing considerations associated with refueling, e.g., nor different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.</li> <li>Burnable poison rods will be used in reload fuel.</li> <li>The number of fuel assemblies (a) in the core and (b) in the spent fue storage pool. a) 177 b) 168</li> </ol>
<ol> <li>Will refueling or resumption of operation thereafter require a technical specification change or other license amendment? If answer is yes, what, in general, will there be? If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)?</li> <li>Yes, some software changes to the Core Protection Calculators will be made, and this will cause some technical specification changes.</li> <li>Scheduled date(s) for submitting proposed licensing action and supporting information. February 1985</li> <li>Important licensing considerations associated with refueling, e.g., nor different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.</li> <li>Burnable poison rods will be used in reload fuel.</li> <li>The number of fuel assemblies (a) in the core and (b) in the spent fuel</li> </ol>
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<ul> <li>be made, and this will cause some technical specification changes.</li> <li>5. Scheduled date(s) for submitting proposed licensing action and supporting information. February 1985</li> <li>6. Important licensing considerations associated with refueling, e.g., nor different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.</li> <li>Burnable poison rods will be used in reload fuel.</li> <li>7. The number of fuel assemblies (a) in the core and (b) in the spent fuel</li> </ul>
<ul> <li>supporting information. February 1985</li> <li>Important licensing considerations associated with refueling, e.g., nor different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.</li> <li>Burnable poison rods will be used in reload fuel.</li> <li>The number of fuel assemblies (a) in the core and (b) in the spent fuel</li> </ul>
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7. The number of fuel assemblies (a) in the core and (b) in the spent fu
<ol> <li>The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or planned, in number of fuel assemblies.</li> </ol>
present 988 increase size by 0
<ol> <li>The projected date of the last refueling that can be discharged to th spent fuel pool assuming the present licensed capacity.</li> </ol>
DATE: 2003



#### ARKANSAS POWER & LIGHT COMPANY

POST OFFICE BOX 551 LITTLE ROCK, ARKANSAS 72203 (501) 371-4000 October 15, 1984

#### 2CAN1Ø84Ø4

Mr. Harold S. Bassett, Director Division of Data Automation and Management Information Office of Resource Management U. S. Nuclear Regulatory Commission Washington, D. C. 20555

SUBJECT: Arkansas Nuclear One - Unit 2

Docket No. 50-368 License No. NPF-6

Monthly Operating Report

(File: 2-0520.1)

#### Gentlemen:

Attached is the NRC Monthly Operating Report for September 1984 for Arkansas Nuclear One - Unit 2.

Very truly yours,

d. Ted Enos

Manager, Licensing

JTE: SAB: ac

Attachment

cc: Mr. Robert D. Martin
Regional Administrator
U. S. Nuclear Regulatory Commission
Region IV
611 Ryan Plaza Drive, Suite 1000
Arlington, TX 76011

Mr. Richard C. DeYoung Office of Inspection and Enforcement U. S. Nuclear Regulatory Commission Washington, DC 20555

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