

UNITED STATES NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

April 23, 1996

Mr. Leon R. Eliason Chief Nuclear Officer & President-Nuclear Business Unit Public Service Electric & Gas Company Post Office Box 236 Hancocks Bridge, NJ 08038

SUBJECT: NRC STAFF'S EVALUATION OF THE INDIVIDUAL PLANT EXAMINATION (IPE)

SUBMITTAL (NLR-N94070) HOPE CREEK GENERATING STATION (TAC NO.

M74421)

Dear Mr. Eliason:

By letter dated May 31, 1994, and supplemented November 6, 1995, you responded to Generic Letter (GL) 88-20, "Individual Plant Examinations for Severe Accident Vulnerabilities," and Supplements 1, 2 and 3, thereto. With the assistance of our contractors, we have completed our review of the IPE submittal for internal events and internal flooding. The evaluation package consists of:

- The Staff Evaluation (SE) (Enclosure 1)
- The contractor's Technical Evaluation Reports (TERs) for the frontend, back-end, and human reliability analysis reviews (Enclosures 2, 3, and 4)

The Hope Creek IPE submittal did not identify any severe accident vulnerabilities associated with either core damage or poor containment performance. We noted that as a result of the IPE, you implemented a number of procedural enhancements and hardware modifications which were reflected in your core damage frequency (CDF) estimate of 4.6 E-5 per reactor year from internally initiated events, including internal flooding. You subsequently advised us that you had upgraded the PRA model, which reduced the estimated CDF to 1.29 E-5 per reactor year. While we commend your reassessment of the capability of the plant to cope with postulated severe accidents, our evaluation was based on your May 31, 1994 submittal.

During the performance of the IPE, transients involving heating, ventilation and air conditioning (HVAC) failures were determined to contribute inordinately to the CDF. It is not clear from the submittal if the Hope Creek analysis considered such items as availability of portable fans, ability to power fans during any and all applicable sequences, indication of switchgear room heatup and available versus needed time, operator training (since new procedure, unfamiliarity exists). It is recommended that the analysis be reexamined to verify these calculated CDFs. We do not consider this finding to have an effect on the overall validity of the IPE.

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Based on our review of the Hope Creek IPE submittal and associated documentation, we conclude that you have fully met the intent of Generic Letter 88-20.

Generic Letter 88-20 proposed that licensees use their IPE submittal to address, among other safety issues, Unresolved Safety Issue (USI) A-45, "Shutdown Decay Heat Removal Requirements." As discussed in the SE and the Front-End TER, this issue is adequately resolved for the Hope Creek Generating Station. You had also proposed to resolve Generic Safety Issue (GSI) 105. "Intersystem LOCA Outside Containment." You estimated the CDF from interfacing systems loss of coolant accidents was only 1.7E-9 per reactor year. This generic issue is also adequately resolved for the Hope Creek Generating Station.

> Sincerely. David H. Jaffe, Senior Project Manager Project Directorate I-2 Division of Reactor Projects - I/II Office of Nuclear Reactor Regulation

Docket No. 50-354

Enclosures: 1. Staff Evaluation

2. TER (Front-End)

3. TER (Back-End)

4. TER (Human Reliability Analysis)

cc w/encl 1: See next page

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Docket File

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ACRS

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NAME	Mo'Brien	RClark	DJane	JStolz for	
DATE	4/22/96	04/18/96	4 1296	4/123/96	

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Sincerely,

David H. Jaffe, Senior Project Manager

Project Directorate I-2

Division of Reactor Projects - I/II Office of Nuclear Reactor Regulation

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cc w/encl 1: See next page

cc:

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