NRC Form 306 [19-93] LICENSEE EVENT REPORT (LER)												U.S. NUCLEAR REGULATORY COMMISSION APPROVED OMB NO. 3150-0104 EXP.RES. 8/31/85									
FACILITY	NAME (1	1	-		dament						-		DOCKET NUMBER	(2)		PAC	E (3)				
- ACILII					mar		**						0 15 10 10	1012	1. 1.	1 OF	ala				
TITLE (4						TO INE		SIV's	DRIF	TING	CLOSE)		1-13-	16 1 6		1012				
EVE	ENT DATE	(5)	T		L	ER NUMBER (8)	RE	PORT DA	TE (7)		OTHER	FACILITIES INVO	L/ED (8)							
MONTH DAY YEAR		R	EAR SEQUENTIAL NUMBER		REVISION	MONTH DAY		YEAR	FACILITY N		MES	DOCKET	CKET NUMBER(S)								
			+			- AUMBER	NOWHER							0 5	1010	101	1.1				
0 1	1 9	8	5	8 5	-	001	-	0 2	1 8	8 5				0 5	10 10	161					
OPE	RATING		T	HIS RE	PORT	Name and Address of the Owner, where the Owner, which is the Owner, wh	D PURSUANT	TO THE R	EQUIREM	ENTS OF	0 CFR 8: /	Check one or more	or the following) (1	1)							
MODE (8)			1	20.402(b)				20.406(c)			x	50,73(a)(2)(iv)		73.71(b)							
POWER			50	20.496(a)(1)(i)			50.38(c)(1)				50 73(a)(2)(v)			73.71(c)							
LEVEL (10) OL 715			01.715 20.40%(a)(1)(iii) 20.40%(a)(1)(iii) 20.40%(a)(1)(iv)			50.38(c)(21 50.73(a)(2)(i) 50.73(a)(2)(ii)				50.73(a)(2)(vii) 50.73(a)(2)(viii) 50.73(a)(2)(viii)		De	OTHER (Specify in Asbelow and in Text, NR 366A)								
				20	406 1)(1)(v)		50.73(a)(2)(iii)			50.73(a)(2)(x)									
								LICENCEE	CONTACT	FOR THIS	LER (12)										
NAME													AREA CODE	TELEPHONE NUMBER							
													AREA CODE								
T. 1	E1	ton	. A	cti	ng	Superir						D IN THIS REPO	9 1 12 RT (13)	316	17 F	17 18	1511				
CAUSE SYSTEM COMP		OMPONEN'		ENT MANUFACTURER					CAUSE	SYSTEM	COMPONENT	MANUFAC- TURER		PROS							
			_	L								111	111								
	1	1	1	1		1 1 1						1 1 1	111								
					_	SUPPLEME	NTAL REPORT	T EXPECT!	ED (14)				1		MONTH	DAY	YEAR				
				-		22.00							SUBMISSI	ON							

On 01/19/85 during performance of the "MAIN STEAM LINE ISOLATION VALVE TRIP TEST" procedure (HNP-2-3111), the "A" inboard MSIV (2B21-F022A) failed to operate within the time limits of Tech. Specs. sections 4.4.7, 4.6.3.3, and item A.1 of Tech. Specs. Table 3.6.3-1. Plant personnel then cycled the "A" MSIV repeatedly to see its time would change such that it would meet the Tech. Specs. requirement. At approximately 0848 CST, during this cycling, the inboard MSIV's drifted to less than 90% open, resulting in an unplanned Reactor Scram.

Plant personnel performed an investigation, and determined that the continuous cyclings of the "A" imboard MSIV resulted in a high rate of charging flow (i.e., greater than or equal to 30 SCFM) to the MSIV's accumulator which caused isolation of the drywell pneumatic system supply valves (2P70-F004 and F005) — these valves isolate when the supply flow rate is greater than or equal 30 SCFM for (2) two minutes. When the drywell pneumatic system supply valves isolated, the MSIV's started drifting closed (due to their accumulators not being charged up and being isolated from their supply).

Plant personnel reviewed HNP-2-3111, and determined that personnel could use it to cycle all of the inboard MSIV's in sequential order or to cycle an inboard MSIV more than once. Thus, by using the procedure, plant personnel could place a greater than or equal to 30 SCFM drain on the drywell pneumatic system supply for two minutes. Thus, this event was the result of procedure inadequacy. This does not affect the outboard MSIVs because they are not fed by the drywell pneumatic system.

HNP-2-3111 is being revised to add a "Note" to allow a two (2) minute wait between operating MSIV's sequentially or cycling MSIV's to prevent a high flow isolation of the drywell pneumatic system.

YES (If yes, complete EXPECTED SUBMISSION DATE)

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (18)

	Fu		
127			

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104 EXPIRES: 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)	DOCKET NUMBER (2)					LER NUMBER (6)							
			YEAR	I	SEQUENTIAL RE			REV	ISION					
EDWIN I. HATCH, UNIT II	0 5 0 0 0 3 6	6	8 5	-	0 0	1	-	0	10	0	12	OF	0	2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

This 30 day report is required by 10 CFR 50.73(a)(2)(iv) since this unplanned scram was an unplanned actuation of an ESF.

On 01/19/85 during performance of the "MAIN STEAM LINE ISOLATION VALVE TRIP TEST" procedure (HNP-2-3111), the "A" inboard MSIV (2B21-F022A) failed to operate within the time limits of Tech. Specs. sections 4.4.7, 4.6.3.3, and item A.1 of Tech. Specs. Table 3.6.3-1. Plant personnel then cycled the "A" MSIV repeatedly to see its time would change such that it would meet the Tech. Specs. requirement. At approximately 0848 CST, during this cycling, the inboard MSIV's drifted to less than 90% open, resulting in an unplanned Reactor Scram.

Plant personnel performed an investigation, and determined that the continuous cyclings of the "A" inboard MSIV resulted in a high rate of charging flow (i.e., greater than or equal to 30 SCFM) to the MSIV's accumulator which caused isolation of the drywell pneumatic system supply valves (2P70-F004 and F005) — these valves isolate when the supply flow rate is greater than or equal 30 SCFM for (2) two minutes. When the drywell pneumatic system supply valves isolated, the MSIV's started drifting closed (due to their accumulators not being charged up and being isolated from their supply).

Plant personnel reviewed HNP-2-3111, and determined that personnel could use it to cycle all of the inboard MSIV's in sequential order or to cycle an inboard MSIV more than once. Thus, by using the procedure, plant personnel could place a greater than or equal to 30 SCFM drain on the drywell pneumatic system supply for two minutes. Thus, this event was the result of procedure inadequacy. This does not affect the outboard MSIVs because they are not fed by the drywell pneumatic system.

HNP-2-3111 is being revised to add a "Note" to allow a two (2) minute wait between operating MSIV's sequentially or cycling MSIV's to prevent a high flow isolation of the drywell pneumatic system.

Georgia Power Company Post Office Box 439 Baxley, Georgia 31513 Telephone 912 367-7781 912 537-9444



Edwin I. Hatch Nuclear Plant

February 18, 1985 GM-85-152

PLANT E. I. HATCH Licensee Event Report Docket No. 50-366

United States Nuclear Regulatory Commission Document Control Desk Washington, D. C. 20555

Attached is Licensee Event Report No. 50-366/1985-001. This report is required by 10CFR 50.73(a)(2)(iv).

H. C. Nix

General Manager

HCN/TLE/Ulz

xc: R. J

R. J. Kelly

R. E. Conway

J. T. Beckham, Jr.

P. D. Rice

K. M. Gillespie

Superintendent of Regulatory Compliance

R. D. Baker

Control Room

Document Control

TEZZ