

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Prairie Island Unit 1	DOCKET NUMBER (2) 0 5 0 0 0 2 8 2	PAGE (3) 1 OF 0 2
--	--------------------------------------	----------------------

TITLE (4)
One Containment Isolation Valve Failed Its Local Leakage Rate Test

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)											
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)									
0	1	2	3	8	5	8	5	0	0	5	0	0	0	2	2	8	5			0 5 0 0 0
																			0 5 0 0 0	

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)

OPERATING MODE (9) N	20.402(b)	20.406(e)	50.73(a)(2)(w)	73.71(b)
POWER LEVEL (10)	20.406(a)(1)(i)	50.38(e)(1)	50.73(a)(2)(v)	73.71(e)
	20.406(a)(1)(ii)	50.38(e)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 388A)
	20.406(a)(1)(iii)	X 50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
	20.406(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
	20.406(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(ix)	

LICENSEE CONTACT FOR THIS LER (12)

NAME Arne A Hunstad, Staff Engineer	TELEPHONE NUMBER 6 1 2 3 8 8 - 1 1 2 1
--	---

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE) NO

EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
	0 3	2 9	8 5

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

During local leakage rate testing at refueling, one containment isolation check valve exhibited leakage in excess of Tech Spec limits. The valve was replaced with a spare. A supplemental report will be submitted when the cause of the failure is determined.

8502270478 850222
PDR ADOCK 05000282
S PDR

IE22 111

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1) Prairie Island Unit 1	DOCKET NUMBER (2) 0 5 0 0 0	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		8 5	- 0 0 5	- 0 0	0 2	OF	0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

During local leakage rate testing at refueling, one containment isolation check valve exhibited leakage in excess of Tech Spec limits. The valve was replaced with a spare. A supplemental report will be submitted when the cause of the failure is determined.



Northern States Power Company

414 Nicollet Mall
Minneapolis, Minnesota 55401
Telephone (612) 330-5500

February 22, 1985

U S Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

PRAIRIE ISLAND NUCLEAR GENERATING PLANT
Docket No. 50-282 License No. DPR-42

One Containment Isolation Valve Failed Its
Local Leakage Rate Test

The License Event Report for this occurrence is attached.

This event was reported via Emergency Notification System per 10 CFR Part 72
on January 23, 1985.

Eugene Eckholt

for David Musolf
Manager - Nuclear Support Services

DMM/EFE/dab

c: Regional Administrator-III, NRC
NRR Project Manager, NRC
Resident Inspector, NRC
MPCA
Attn: J W Ferman

Please replace LER 85-05, dated
2/21/85 (which was sent without
a signature on the cover letter),
with the attached.

Attachment

IE22
1/1