

Memo to File

APR 11 1974

Re: Jersey Central Power & Light Co. Reply of 3/13/74 to
Inspection at Oyster Creek on 12/27-28/73.

I called Don Ross on 3/15/74 to clarify his objections to my report:

Item 1: "Stack gas rate" was incorrectly described as "off-gas rate".
24uCi/sec should have been 24 mCi/sec.

Item 2: H. P. Tech shift assignments have been discussed with the
union. One tech will be assigned to each of the two off-shifts.

Item 3: I quoted my written notes on my phone conversation with Ross
on 1/3/74 in which he stated "three-page policy statement will be
issued next week". He agreed that he had said this and therefore
understood my statement that "the program would go into effect the
week of January 7." This was not his intent, however. The program
was put into effect on 3/15/74. I told him I would delete the date
from my report.

Item 4: My word "submitted" was incorrect. It should have been
"drafted". I had understood Don Reeves to say "submitted" at the
close-out meeting and my notes so reflect. I agreed to change my
report.

Section 6, paragraph 1, and last statement on page 9:

I had quoted from my notes from a conversation with Jin Maloney,
Operations Supervisor. Ross stated that Maloney had perhaps been
somewhat inaccurate in his statement to me and that Ross would speak
to him. Their goal is zero release after the new Radwaste facility
is completed. They intend to comply with 10 CFR 50, Appendix I.
My statement about the laundry tank being the only source of water
discharged should have added "released in many weeks". In fact,
some additional water has been released recently.

John Mann

John Mann
Radiation Specialist

B/578

To: James P. O'Reilly
Directorate of Regulatory Operations
Region I
631 Park Avenue
King of Prussia, Pennsylvania 19406

From: Jersey Central Power & Light Company
Oyster Creek Nuclear Generating Station Docket #50-219
Forked River, New Jersey 08731

Subject: Abnormal Occurrence Report No. 50-219/74/ 25

The following is a preliminary report being submitted
in compliance with the Technical Specifications
paragraph 6.6.2.

Preliminary Approval:

J. T. Carroll, Jr. 4/10/74
J. T. Carroll, Jr. Date

cc: Mr. A. Giambusso

~~2344114444~~ (488)

B/579

Initial Telephone
Report Date: 4/9/74

Date of
Occurrence: 4/9/74

Initial Written
Report Date: 4/10/74

Time of
Occurrence: 1040

OYSTER CREEK NUCLEAR GENERATING STATION
FORKED RIVER, NEW JERSEY 08731

Abnormal Occurrence
Report No. 50-219/74/25

IDENTIFICATION
OF OCCURRENCE:

Violation of the Technical Specifications, paragraph 3.5.A.1,
loss of primary containment integrity with the reactor criti-
cal and the reactor water temperature greater than 212°F.

This event is considered to be an abnormal occurrence as de-
fined in the Technical Specifications, paragraph 1.15B.

CONDITIONS PRIOR
TO OCCURRENCE:

<input checked="" type="checkbox"/> Steady State Power	<input type="checkbox"/> Routine Shutdown
<input type="checkbox"/> Hot Standby	<input type="checkbox"/> Operation
<input type="checkbox"/> Cold Shutdown	<input type="checkbox"/> Load Changes During
<input type="checkbox"/> Refueling Shutdown	<input type="checkbox"/> Routine Power Operation
<input type="checkbox"/> Routine Startup	<input type="checkbox"/> Other (Specify)
<input type="checkbox"/> Operation	

The major plant parameters at the time of the event were as
follows:

Power: Reactor, 1849 MWt
Electrical, 642 MWe
Flow: Recirc., 61×10^6 lbm/hr
Feedwater, 6.9×10^6 lbm/hr
Stack Gas: 34,895 μ Ci/sec

DESCRIPTION
OF OCCURRENCE:

At 1040 on April 9, 1974, a local leak rate test on the
reactor building to torus vacuum breakers was commenced and
it was discovered that pressure could not be placed between
V-26-15 and 16, the check and butterfly isolation valves. A
check of leakage of drywell atmosphere to the reactor building
was indicated when a plastic bag was

the bag filled with air indicating that the outside (check) valve was leaking. The check valve was cycled several times and then forced to seat. Air was again admitted between the valves and no leakage was detected through the check valve, but the pressure would not exceed 17.5 psig. This indicated that the butterfly valve, V-26-16, was leaking. Upon inspection of V-26-16, it was found that the valve had not been in the fully closed position. The valve was fully closed manually and a successful leak test was performed.

The butterfly valve, V-26-16, was made inoperable, as permitted by Technical Specification 3.5.A.5.

APPARENT CAUSE
OF OCCURRENCE:

<input type="checkbox"/> Design	<input type="checkbox"/> Procedure
<input type="checkbox"/> Manufacture	<input type="checkbox"/> Unusual Service Condition
<input type="checkbox"/> Installation/ Construction	<input type="checkbox"/> Inc. Environmental
<input type="checkbox"/> Operator	<input checked="" type="checkbox"/> Component Failure
	<input type="checkbox"/> Other (Specify)

The cause for valves V-26-15 and V-26-16 not to seal properly is not known at this time.

ANALYSIS OF
OCCURRENCE:

An initial analysis of the data indicates that under accident conditions in the drywell (35 psig) the leakage rate through V-26-15 and V-26-16 would have been approximately 20% to 40% of the allowable leakage rate from the primary containment. Additional investigation is continuing.

CORRECTIVE
ACTION:

At 1408, a reactor shutdown was commenced. Both valves were subsequently properly seated, successfully leak tested, and made inoperable. At 1443, the reactor shutdown was terminated and power was increased to the initial value.

FAILURE DATA:

History of torus to reactor building vacuum breakers:

- 11/23/70 - V-26-18 failed to open during operability surveillance
- 12/18/70 - V-26-16 and V-26-18 failed to open during operability surveillance
- 1/12/71 - V-26-18 failed leakage rate test
- 1/13/71 - V-26-18 linkage tightened one turn, passed leakage test but valve would not open
- 1/14/71 - V-26-18 adjusted controller, passed leakage test and passed operability test
- 2/17/71 - V-26-16 and V-26-18 changed seats and both passed leakage tests
- 5/3/73 - V-26-18 failed leakage test. Linkage adjusted and valve passed leakage and operability tests.

Prepared by:

Karl P. G. Fickenscherf

Date:

4/10/74

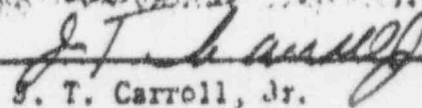
To: James P. O'Reilly
Directorate of Regulatory Operations
Region I
631 Park Avenue
King of Prussia, Pennsylvania 19406

From: Jersey Central Power & Light Company
Oyster Creek Nuclear Generating Station Docket #50-219
Forked River, New Jersey 08731

Subject: Abnormal Occurrence Report No. 50-219/74/ 24

The following is a preliminary report being submitted
in compliance with the Technical Specifications
paragraph 6.6.2.

Preliminary Approval:

 4/10/74
S. T. Carroll, Jr. Date

cc: Mr. A. Giambusso

~~8304110050~~ (488)

B/580

Report Date: 4/2/74 Occurrence: 4/2/74
 Initial Written Report Date: 4/10/74 Time of Occurrence: 1000

OYSTER CREEK NUCLEAR GENERATING STATION
 FORKED RIVER, NEW JERSEY 08731

Abnormal Occurrence
 Report No. 50-219/74/24

IDENTIFICATION
 OF OCCURRENCE:

Violation of the Technical Specifications, paragraph 4.6.B.1.g, in that the stack gas particulate filter in service from March 28, 1974 to March 31, 1974 was not analyzed for gross β , gross α , and gross γ .

This event is considered to be an abnormal occurrence as defined in the Technical Specifications, paragraph 1.15G.

CONDITIONS PRIOR
 TO OCCURRENCE:

- | | | | |
|-------------------------------------|--------------------|--------------------------|-------------------------|
| <input checked="" type="checkbox"/> | Steady State Power | <input type="checkbox"/> | Routine Shutdown |
| <input type="checkbox"/> | Hot Standby | <input type="checkbox"/> | Operation |
| <input type="checkbox"/> | Cold Shutdown | <input type="checkbox"/> | Load Changes During |
| <input type="checkbox"/> | Refueling Shutdown | <input type="checkbox"/> | Routine Power Operation |
| <input type="checkbox"/> | Routine Startup | <input type="checkbox"/> | Other (Specify) |
| <input type="checkbox"/> | Operation | | |

The major plant parameters at the time of the event were as follows:

Power: Reactor, 1806 MWt
 Electrical, 632 MWe
 Flow: Recirc., 57.2×10^6 lb/hr
 Feedwater, 6.735×10^6 lb/hr
 Stack Gas: 35,000 μ Ci/sec

DESCRIPTION
 OF OCCURRENCE:

A stack gas particulate filter, installed at 0847 on March 28, 1974 and removed at 0854 on March 31, 1974, was not counted for gross α , gross β , and gross γ within one week of removal.

APPARENT CAUSE
OF OCCURRENCE:

<input type="checkbox"/> Design	<input type="checkbox"/> Procedure
<input type="checkbox"/> Manufacture	<input type="checkbox"/> Unusual Service Condition
<input type="checkbox"/> Installation/ Construction	<input type="checkbox"/> Inc. Environmental Component Failure
<input type="checkbox"/> Operator	<input type="checkbox"/> Other (Specify)

Counting of the filter 48 hours after removal was not performed by the technician as required. Investigation into the cause is continuing.

ANALYSIS OF
OCCURRENCE:

The safety significance connected with this occurrence is that any unusually large release of particulate activity during this period might not have been recognized until the monthly composite analyses were complete. This is not a likely possibility as a spectrum analysis was performed to measure releases of short-lived isotopes and if unusually large amounts of activity were present, it would have been readily apparent. In addition, a spectrum analysis of the charcoal filter had also been performed and normal quantities of Iodine were found.

CORRECTIVE
ACTION:

The particulate filter was counted nine days after filter removal upon discovery of the abnormal occurrence. The gross α , gross β , and gross γ values were comparable to samples removed before and after this filter. This would indicate that there was little or no change in plant gaseous effluents and that the plant was operating with releases less than 4% of the Technical

Specification limit of 4 $\mu\text{Ci}/\text{sec}$ of Iodine and particulates
having half-lives >8 days.

Prepared by:

Robert L. Stouckous

Date:

4/10/74