OPERATING DATA REPORT

DOCKET NO. 50-247

DATE 2-8-85

COMPLETED BY M. Blatt
TELEPHONE (914) 526-2127

. Unit Name: Indian Point Unit #2	Notes
Reporting Period: January 1985	
Licensed Thermal Power (MWt): 2758	
. Nameplate Rating (Gross MWe):	The second of th
Design Electrical Rating (Net MWe): 873	
Maximum Dependable Capacity (Gross MWe): 900	
Maximum Dependable Capacity (Net MWe): 864	
. If Changes Occur in Capacity Ratings (Items Number 3 Th	hrough 7) Since Last Report, Give Reasons:
None	
None None Power Level To Which Restricted, If Any (Net MWe):	None .

	This Month	Yrto-Date	Cumulative
	744	744	9.2833
11. Hours In Reporting Period	739.92	739.92	61405.64
12. Number Of Hours Reactor Was Critical	0	0	2345.70
13. Reactor Reserve Shutdown Hours	717.35	717.35	59467.73
14. Hours Generator On-Line	0	0	0
15. Unit Reserve Shutdown Hours	19 589 19	19 589 19	154663294
16. Gross Thermal Energy Generated (MWH)	625450	625450	47943066
17. Gross Electrical Energy Generated (MWH)	602906	6029.06	45716813
18. Net Electrical Energy Generated (MWH)	. 96.4	96.4	64.1
19. Unit Service Factor	96.4	96.4	64.1
20. Unit Availability Factor	93.8	93.8	57.3
21. Unit Capacity Factor (Using MDC Net)	92.8	92.8	56.4
22. Unit Capacity Factor (Using DER Net)	3.6	3.6	9.6
23. Unit Forced Outage Rate 24. Shutdowns Scheduled Over Next 6 Months (Ty	Secretaria dell'Arte dell'		

25. If Shut Down At End Of Report Period, Estimated Date of Startup:

26. Units In Test Status (Prior to Commercial Operation):

Forecast

Achieved

INITIAL CRITICALITY
INITIAL ELECTRICITY
COMMERCIAL OPERATION

8502270349 850131 PDR ADOCK 05000247 R PDR IE24

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO.	50-247				
UNIT	I.P. Unit #2				
DATE	2/8/85				
COMPLETED BY	M. Blatt				
TELEPHONE	914-526-5127				

AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY - 17	AVERAGE DAILY POWER LEVEL (MWe-Net) 854
459	18	856
846	19	853
848	20	854
852	21	855
852	22	852
848	23	856
854	. 24	852
855	25	855
852	26	852
853	27	853
853	28	855
854	29	852
827	30	855
839	31	853
847		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net f. . each day in the reporting month. Compute to the nearest whole megawatt.

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH

JANUARY 1985

50-247 DOCKET NO. UNITNAME IP Unit #2 DATE 2/8/85 COMPLETED BY M. Blatt TELEPHONE (914) 526-512

No.	Date	Type1	Duration (Hours)	Reason-	Method of Shutting Down Reactor?	Licensee Event Report #	System Code ⁴	Component Code5	Cause & Corrective Action to Prevent Recurrence
7	841219	F	26-65	A	3	84-025	НА	xxxxxx	Generator Hydrogen Seal Leak Causing Fire
N/A	850114	F	N/A	A	N/A	N/A	EB	XXXXXX	MCC Trip Causing Loss of IRPI and Turbine Runback

F: Forced

S: Scheduled

A-Equipment Failure (Explain) B-Maintenance of Test

C-Refueling

D-Regulatory Restriction

E-Operator Training & License Examination F-Administrative

G-Operational Error (Explain) H-Other (Explain)

Method: 1-Manual

2-Manual Scram.

3-Automatic Scrain. 4-Other (Explain)

Exhibit G - Instructions for Preparation of Data Entry Shects for Licensee Event Report (LER) File (NUREG-01611

Exhibit 1 - Same Source

(1)/77)

Summary of Operating Experience

February 1985

At the beginning of the report period, Indian Point Unit 2 was in hot shutdown to complete repairs to the main generator hydrogen seal. On January 2 the unit was returned to service and reactor power was gradually raised to 100%, which was maintained for the remainder of the report period.

MAJOR SAFETY RELATED CORRECTIVE MAINTENANCE

MWR NO.	SYSTEM	COMPONENT	DATE	WORK PERFORMED
17313	WDS	22 Waste Gas Compressor Breaker	1/09/85	Replaced breaker
17314	cvcs	22 Boric Acid Transfer Pump	1/24/85	Rebuilt pump.
17617	RCS	23 Pressurizer Back-up Heater	1/28/85	Replaced auxiliary switch in feed supply breaker to heater
16954	PSS	#1 Liquid Sample Pump-Primary Sampling System	1/29/85	Replaced pump.
16893	RP	Channel B of In- dependent Turbine Electronic Overspeed Pro- tection System	12/02/84	Replaced solenoid coil
17256	WD	22 Waste Gas Compressor	12/04/84	Rebuilt waste gas compressor,
17391	RM	Rll Radiation Monitor Vapor Containment	12/20/84	Replaced blower.
17977	RM	R11 & R12 Vapor Containment Radiation Monitors	12/22/84	Replaced failed coil in solenoid valve on sample line
17987	ВА	Valve 210 in Reactor Water Makeup System	12/28/84	Replaced valve stem.
17528	SI	21 Safety In- jection System Pump	12/31/84	Replaced pump.
16428	BA	21 Boric Acid Transfer Pump	11/07/84	Replaced pump bearings shaft oil and mechanical seals.
16427	BA	22 Boric Acid Transfer Pump	11/07/84	Replaced pump bearings, shaft oil and mechanical seals.

MAJOR SAFETY RELATED CORRECTIVE MAINTENANCE

MWR NO.	SYSTEM	COMPONENT	DATE	WORK PERFORMED
16701	IA	21 Instrument AJR Compressor Dis- charge Valves IA-3 & IA-3-1	11/13/84	Replaced worn valves
16864	VC	80' Vapor Con- tainment Air- lock Outside Door	11/16/84	Replaced bearings in locking assembly
13779	vc	27 Hot Penetra- tion Air Blower	11/27/84	Replaced motor.
17435	BA	21 Boric Acid Transfer Pump	11/30/84	Replaced pump seal, impeller "O" Ring, gland follower and mechanical seal.

Attachment I

February, 1985
Docket No. 50-247
Ref: Letter to Hartfield/Cahill
1/18/78

Refueling Information Request

- 1. Name of facility Indian Point Unit No. 2
- Scheduled date for next refueling shutdown March 1986.
- Date for restart following refueling Unit made critical October 17, 1984.
- 4. Will refueling or resumption of operation the eafter require a technical specification change or other license amendment?

-No fuel design or core configuration related technical specification change or license amendment is required.

If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Ref. 10 CFR Section 50.59)? If no such review has taken place when is it scheduled?

-The reload fuel design and core configuration has been reviewed in accordance with the requirements of the plant Technical Specifications.

 Scheduled date(s) for submitting proposed licensing action and supporting information.

-Not applicable.

6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.

-None.

- 7. The number of fuel assemblies.
 - a) in the core 193

and

- b) in the spent fuel storage pool 396
- 8. The present licensed spent fuel storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies.

-Maximum storage capacity is 980 (Tech. Spec. Amendment 75). There are no plans or request for any increase.

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.

-Year 1993 with full core discharge capability.

Attachment II

Summary of Operating Experience

December 1984

Following a scheduled two day outage to perform maintenance on the boiler feed pumps during which the plant was maintained in hot shutdown, Indian Point Unit 2 was returned to service on December 2 and power gradually raised to full load conditions.

On December 19 the Unit was removed from service due to a fire that occurred at the generator exciter end seal. The fire resulted from the failure of the main generator hydrogen shaft seal.

On December 28 the RCS was cooled below $350^{\circ}\mathrm{F}$ where it was maintained for the remainder of the report period to facilitate maintenance of the safety injection pumps.

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH

DOCKET NO. _50-247 UNITNAME I. P. Unit No. DATE _1/7/85 M. Blatt COMPLETED BY 914-526-5127 TELEPHONE

No.	Date	Type1	Duration (Hours)	Reason?	Method of Shutting Down Reactor 3	Licensee Event Repurt #	System Code4	Component Code 5	Cause & Corrective Action to Prevent Recurrence
6	841130	s	46.98	В	1	NA	на	zzzzzzz	Misc. Maintenance Continued From November
7	841219	F	289.97	A	3	84-025	НА	xxxxxx	Generator Hydrogen Seal Leak Causing Fire

F: Forced S. Scheduled

Reason:

All quipment Failure (Explain)

B-Maintenance of Test

C.Refueling

D-Regulatory Restriction

F-Operator Training & License Examination

F-Administrative

G-Operational Error (Explain) H-Other (Explain)

Method:

I-Manual

2-Manual Scram.

3-Automatic Scrain.

4-Other (Explain)

Exhibit G - Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

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Exhibit 1 - Same Source

(1/77)

Consolidated Edison Company of New York, Inc. 4 Irving Place, New York, NY 10003
Telephone (212) 460-2533

February 15, 1985

Re: Indian Point Unit No. 2
Docket No. 50-247

Mr. James M. Taylor, Director
Office of Inspection and Enforcement
c/o Distribution Services Branch, DDC, ADM
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Dear Mr. Taylor:

Forwarded herewith is one copy of the Monthly Operating Report for Indian Point Unit No. 2 for the month of January 1985. We are enclosing as Attachment I an update on the refueling information requested in a January 18, 1978 letter from Mr. R. A. Hartfield, Acting Director of the Office of Management Information and Program Control.

Also enclosed as Attachment II are a revised Unit Shutdown and Power Reductions and Summary of Operating Experience pages for the month of December 1984.

John D. Forle

Encl.

cc:

Dr. Thomas E. Murley, Regional Administrator Region I U.S. Nuclear Regulatory Commission 631 Park Ave. King of Prussia, Pa. 19406

Senior Resident Inspector U.S. Nuclear Regulatory Commission P.O. Box 38 Buchanan, New York 10511

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